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Experimental overview of ELM control using external magnetic perturbations



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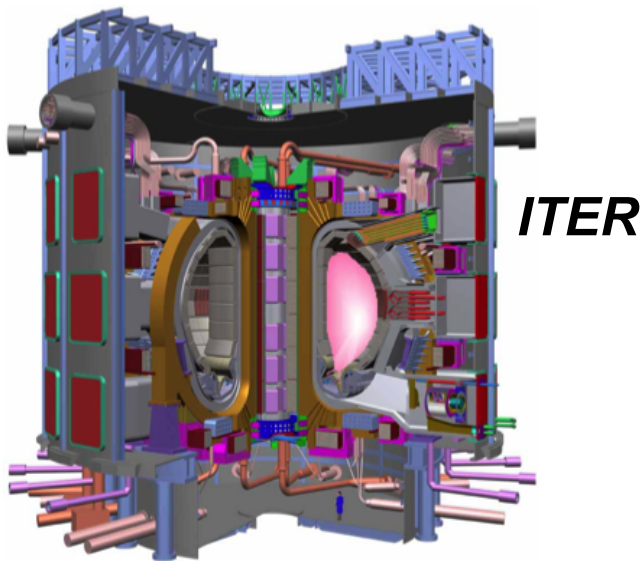
13th International Workshop on Plasma Edge Theory

South Lake Tahoe

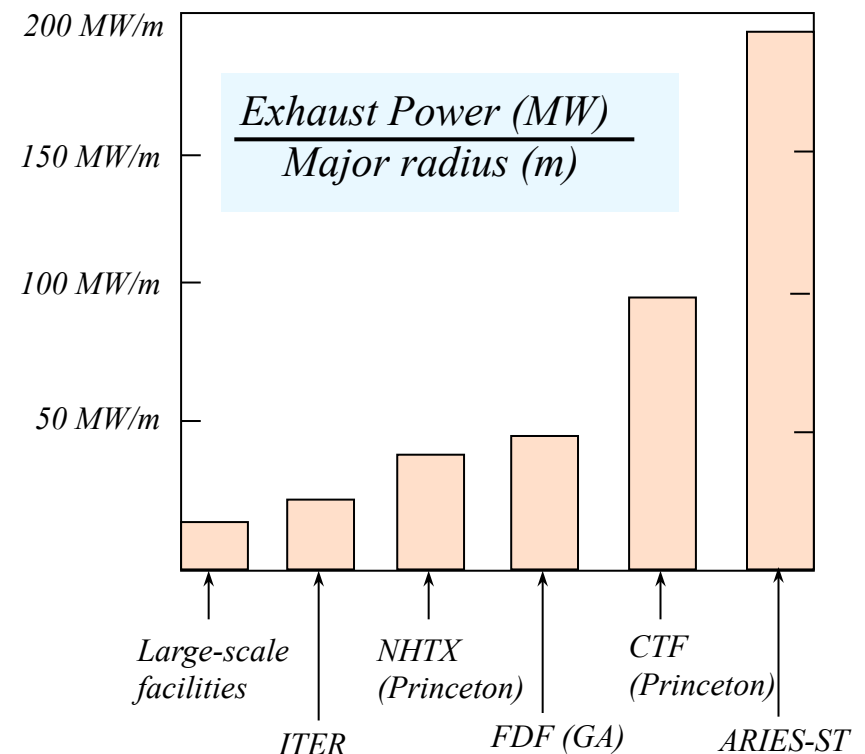
September 19th, 2011

BURNING PROBLEM: Next-generation tokamak reactors desperately need techniques to reduce **heat fluxes**

- Tokamak performance has reached the level where heat flux poses an issue
 - The steady-state divertor heat load in ITER is only marginally manageable
- Transient “bursty” events greatly increase damage to divertor & first wall
 - ITER has no tolerance for ELMs

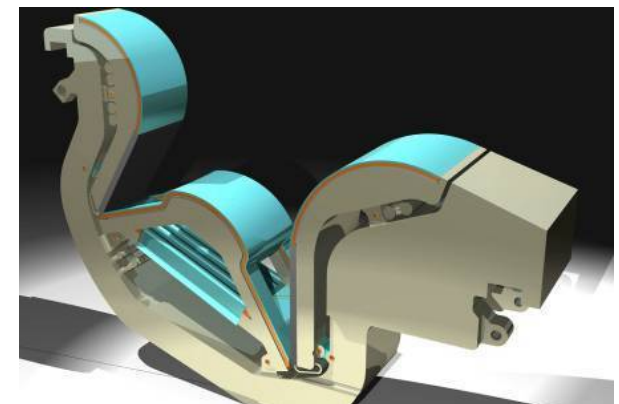
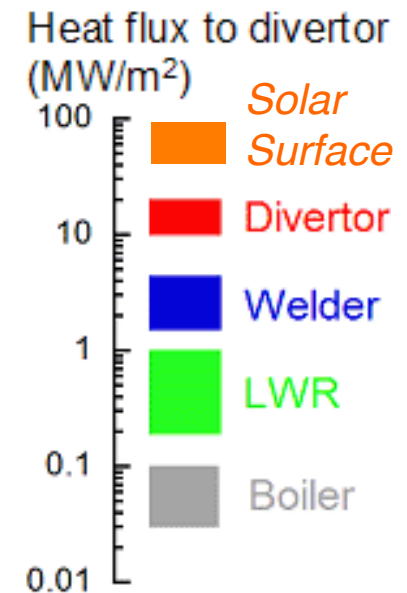


- ***These issues becomes increasingly difficult for next-step devices!***



Exhaust power must be limited to acceptable levels for divertor target plate materials

- **Steady-state limit for continuous heating**
 - Constrained by target plate material phase transition
C (sublimation) or W (melting)
 - Heat flux $Q < 10 \text{ MW/m}^2$
- **Transient limits for bursts of heat more stringent**
 - Constrained by target plate heat conduction
 - Heat impulse $Q \tau^{1/2} < 35\text{-}50 \text{ MJ s}^{1/2}/\text{m}^2$
- **Unfortunately, bursty transport is ubiquitous**
 - Prediction for type-I ELMs on ITER at $\nu^*=0.6$
 - $\Delta W_{\text{ELM}} \sim 20 \text{ MJ}$ [Loarte, 2003]
 - But, limits have decreased as research continues
 - $\Delta W_{\text{ELM}} < 1 \text{ MJ}$ [Federici 2006]
 - $\Delta W_{\text{ELM}} < 0.2 \text{ MJ}$ [Pitts EPS 2011]



ITER divertor target cassette
CFC tiles & W armor

At least three different flavors of ELM control exist

1. ELM suppression

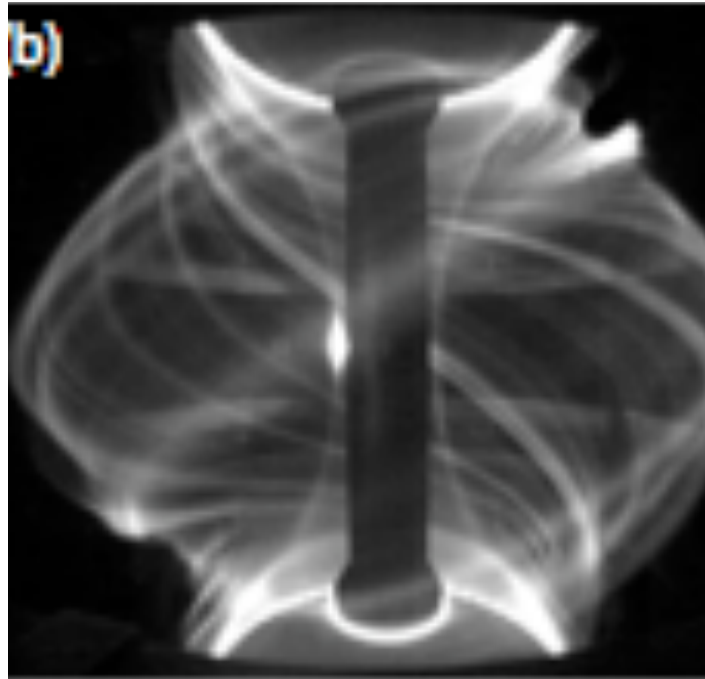
- No discernable ELMs remain
- Low collisionality threshold

2. ELM mitigation

- Small ELMs remain (prob. not type-I)
- High-collisionality threshold?

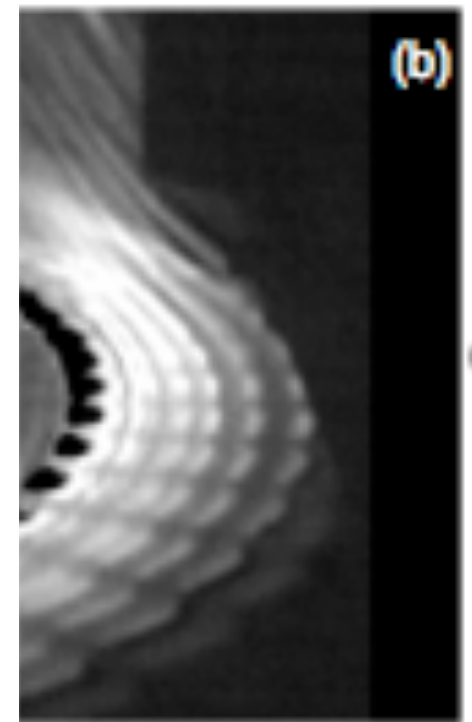
3. ELM triggering

- Used to control impurity transport in ELM-free regimes



*ELM in $D\alpha$ on MAST tokamak
A. Kirk, Plasma Phys. Control. Nucl.
Fusion, **49** 1259 (2004)*

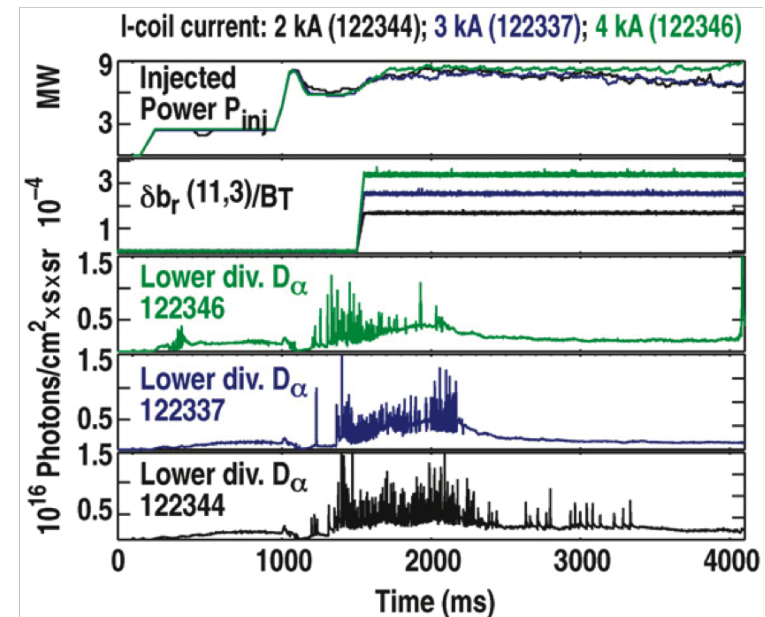
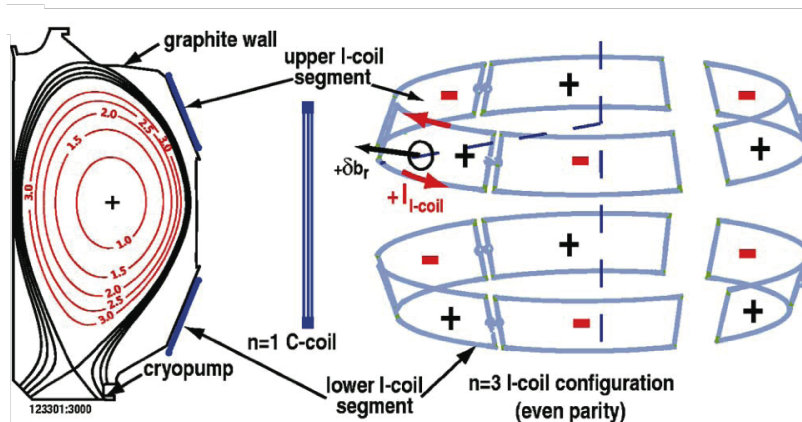
*Recycling profile on
target plates*



DIII-D discovered that non-axisymmetric magnetic perturbations can stabilize ELMs ... and destabilize ELMs

- ELM mitigation first found on DIII-D H-mode plasmas¹

- Using external coils



- Full ELM suppression then discovered at low collisionality^{2,3}

- Threshold in perturbation amplitude

- $\delta B/B \sim 3 \times 10^{-3}$

- Threshold in pitch-resonant field $k_{\parallel} = (m - qn)/qR = 0$

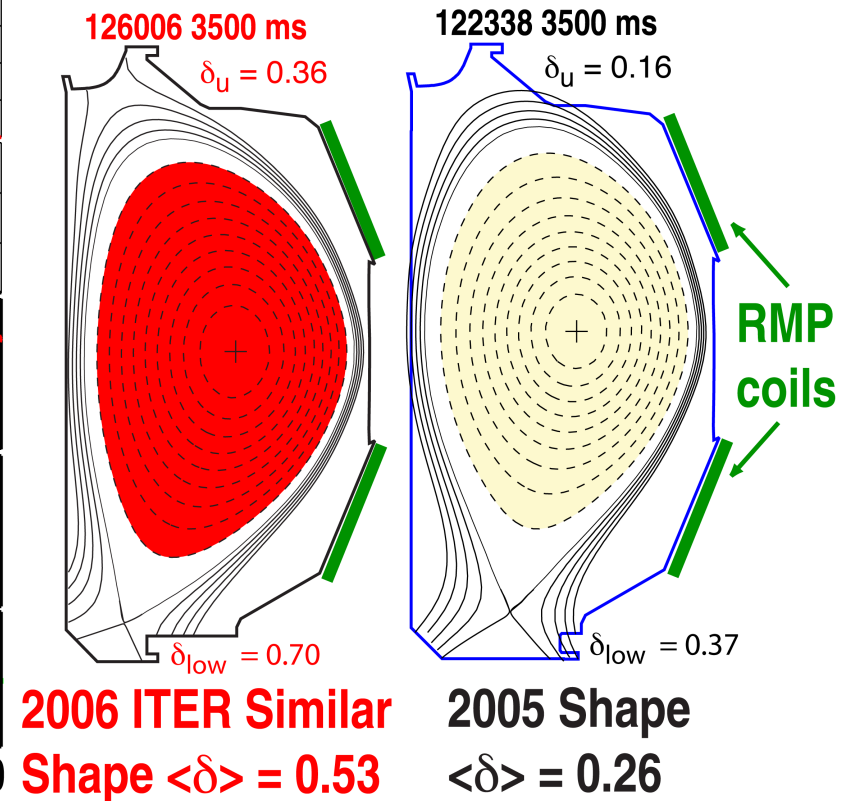
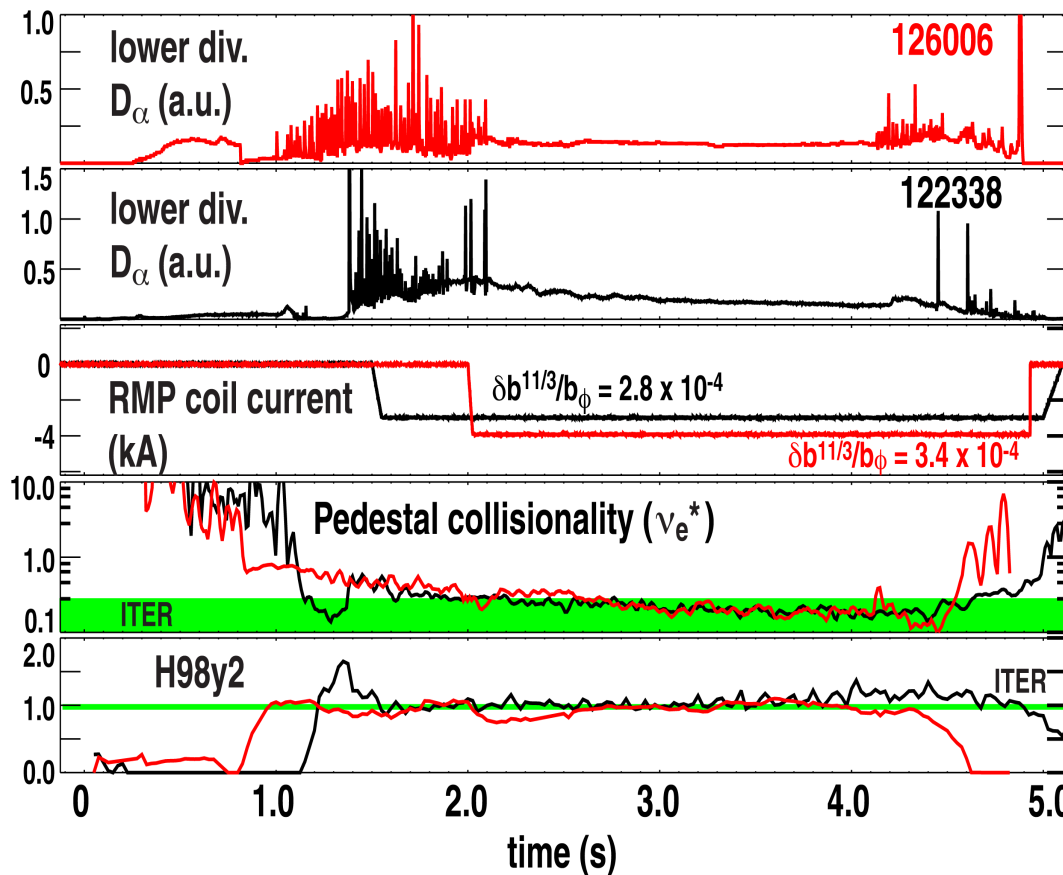
- $\delta B_{m=qn}/B \sim 3 \times 10^{-4}$

¹T.E. Evans, et al., Phys. Rev. Lett. **92** 235002 (2004)

²K.H. Burrell, et al., Plasma Phys. Control. Fusion **47** B37 (2005)

³T.E. Evans, et al., Nature Phys. **2** 419 (2006)

ELMs are Completely Eliminated with RMP Fields while Maintaining the ITER H-mode Confinement Target

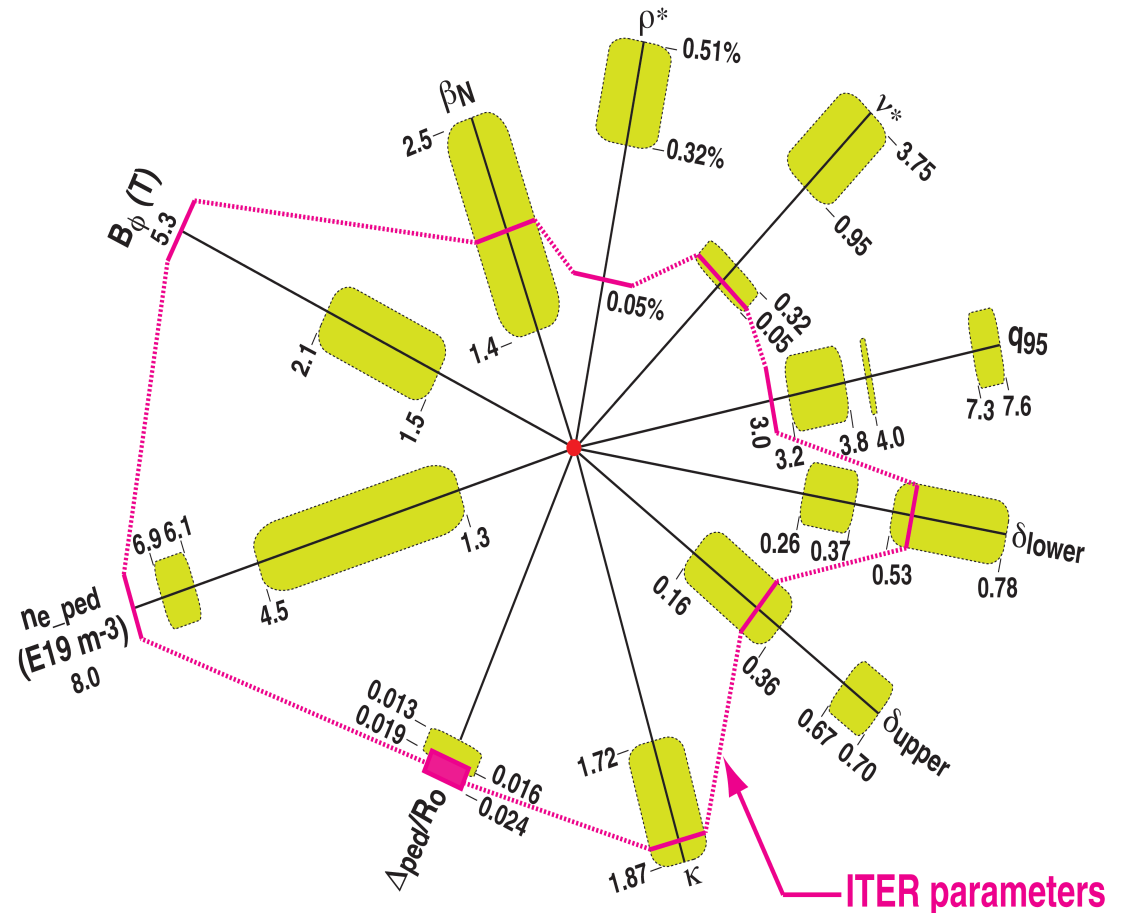


- Full suppression is obtained over a range of plasma shapes at an ITER pedestal collisionality

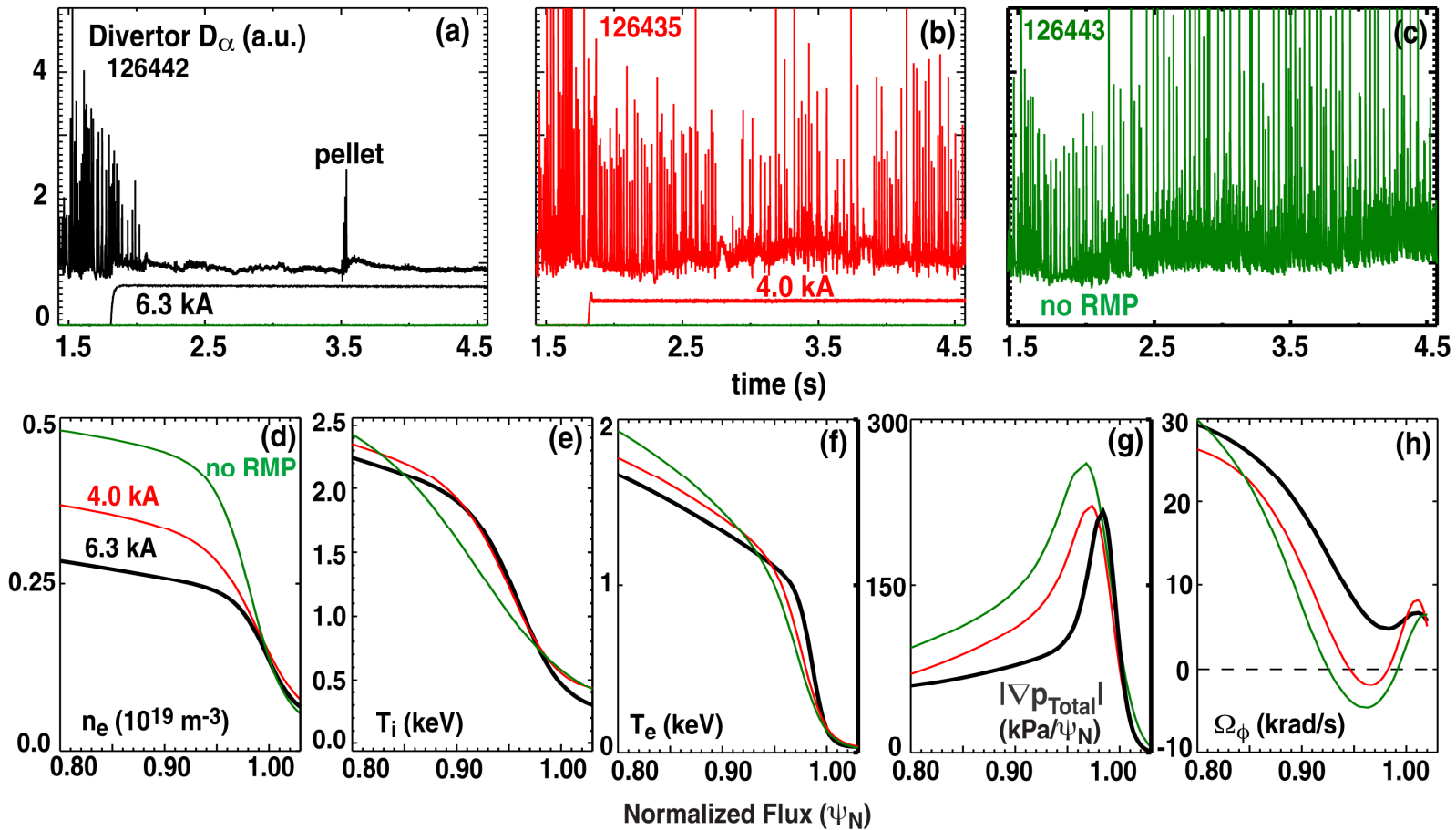
ELM Suppression is Obtained over a Wide Range of DIII-D Operating Parameters

DIII-D ELM suppression operating space

- RMP ELM suppression obtained with all key ITER dimensionless parameters
 - Except for ρ^* and q_{95}
- ITER dimensional parameters such as B_ϕ , n_{e_ped} and are not accessible in DIII-D
- Models are needed to assess scaling to ITER



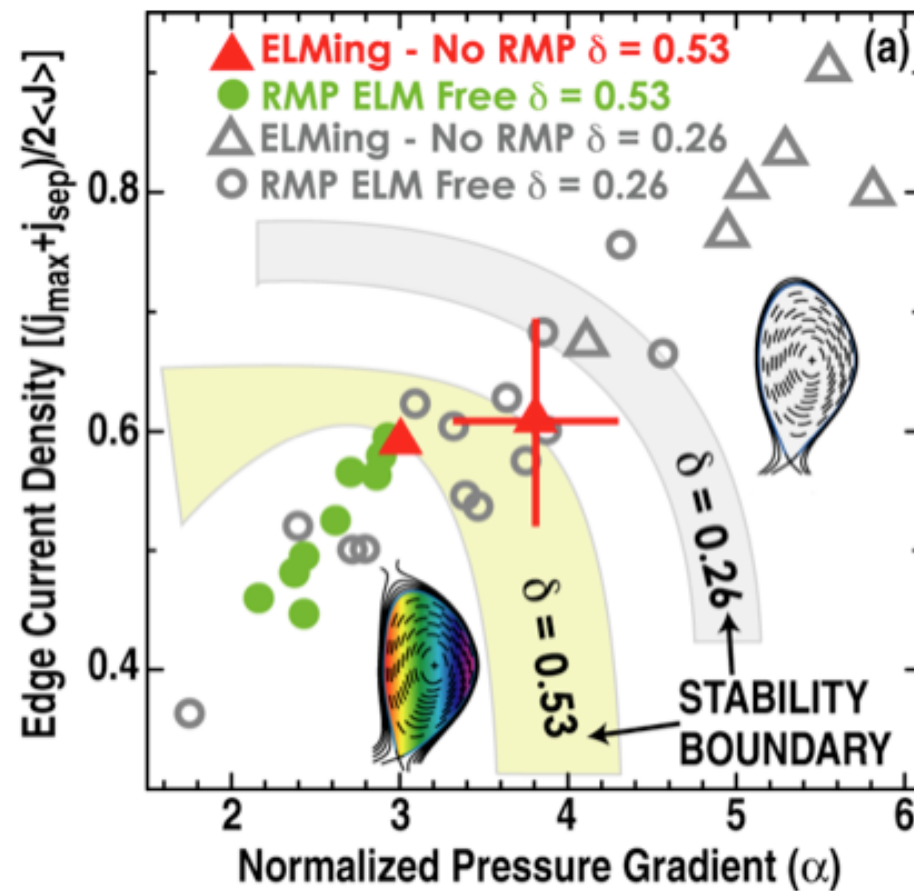
For ELM suppression at low-collisionality, the edge pressure is reduced via increased particle transport



¹T. E. Evans, et al., Nucl. Fusion **48** 024002 (2008)

- Global n_e reduced as RMP increased (even in core)
- T_e and T_i actually increase in transport barrier region!

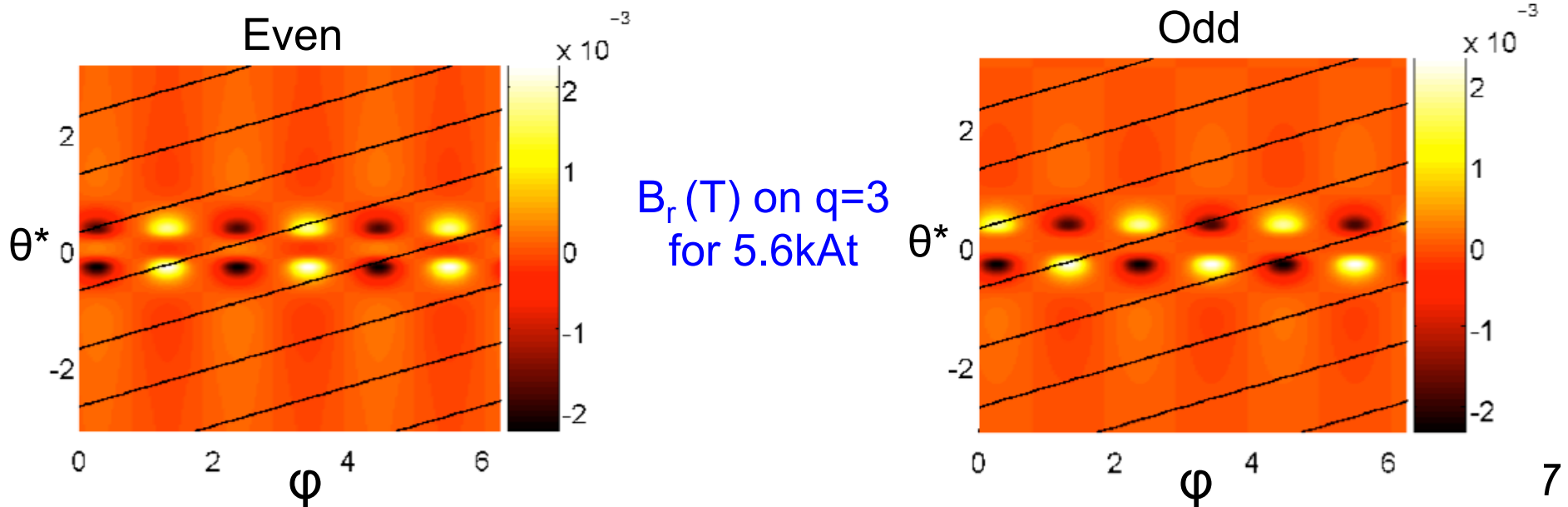
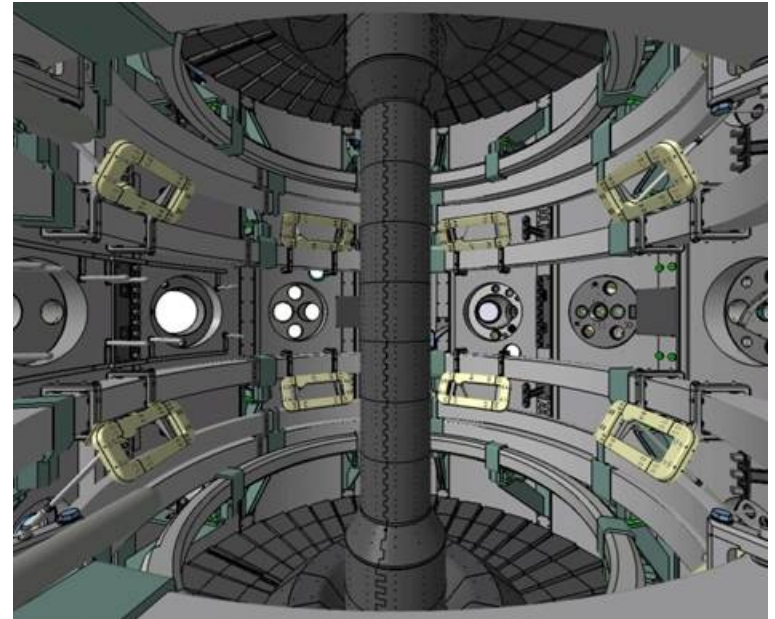
For low collisionality, reduction in edge ∇p & J_{\parallel} stabilizes type-I ELMs



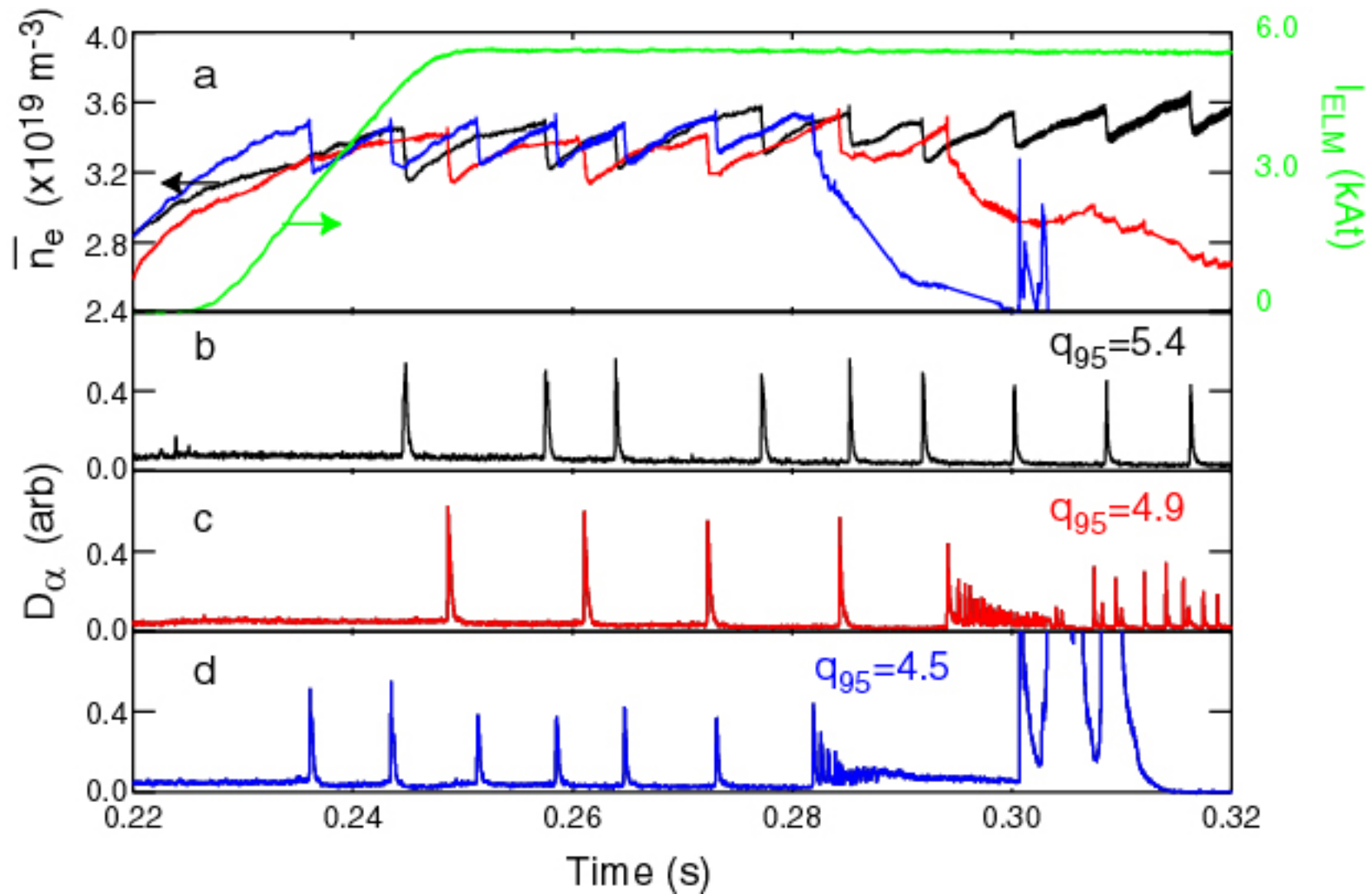
T. H. Osborne, et al., *J. Phys.: Conf. Series A* **123** 012014 (2008)

The MAST ELM control coils

- 2 rows of 6 in-vessel coils, producing $n=3$ perturbations: similar to DIII-D I-coils
- Dimensioned so as to satisfy the Chirikov criterion $\Delta_{\text{Chir}>1} > 0.17$
- Coils can carry up to $5.6\text{kAt} = 1.4\text{kA} \times 4 \text{ turns}$
- Even and Odd configurations are possible
 - Complementary: when one is on resonance, the other is off resonance
 - ⇒ Allows to test for resonant effects

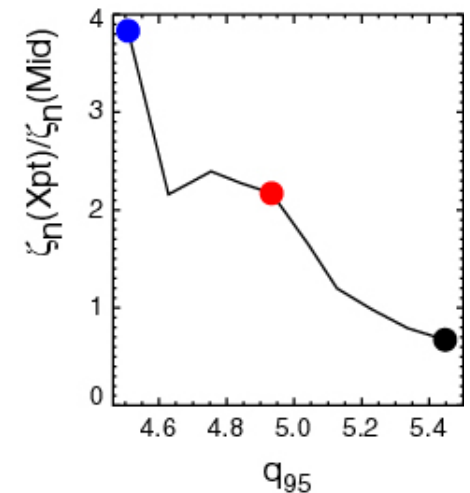
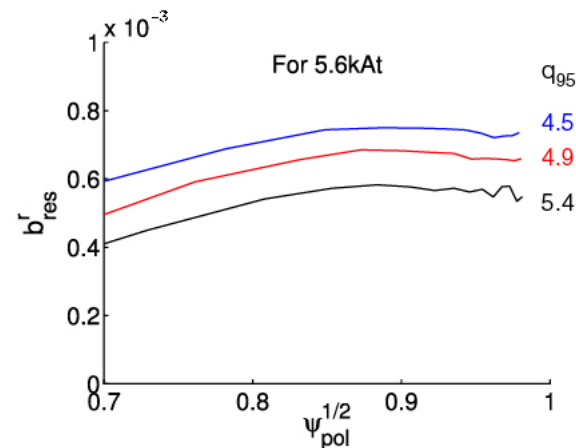
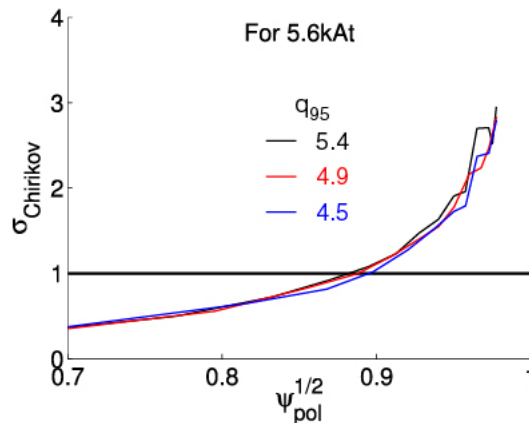
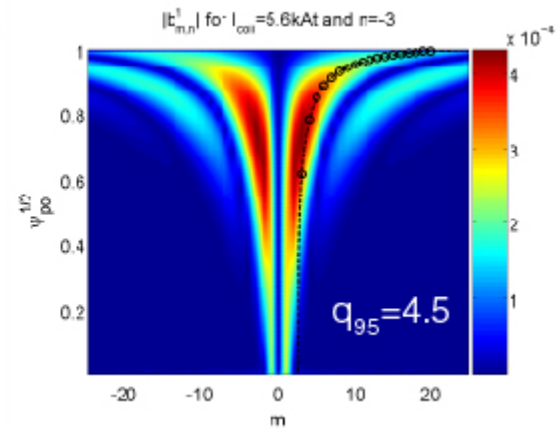
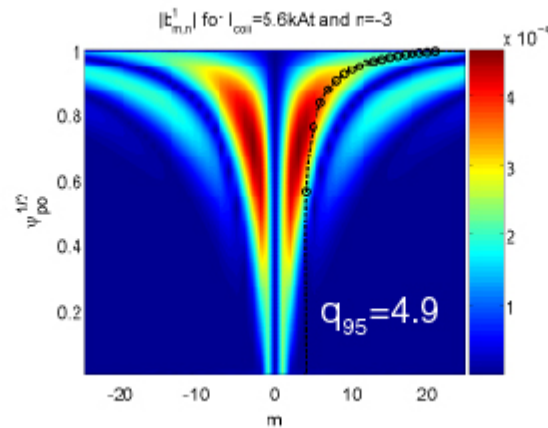
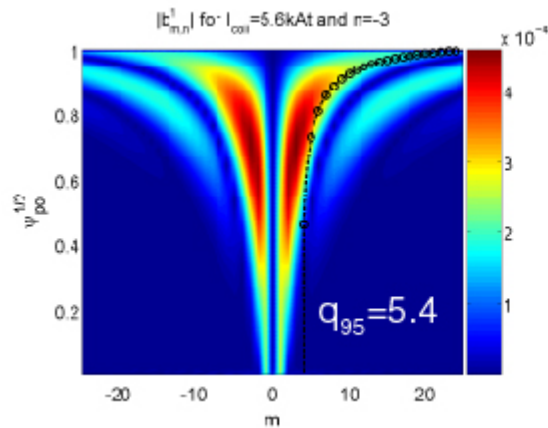


Effect on type I ELMs – q_{95} scan



Back transition to L-mode seen at $q_{95} \sim 4.5$

q₉₅ scan - modelling



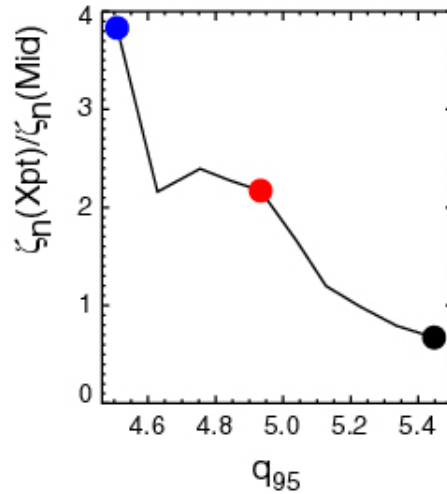
Little effect on $\Delta_{\psi_{Nochir>1}} (\approx 0.19)$
– since decreasing q_{95} decreases island overlap due to changes in shear

Field alignment improves
→ increase in resonant field (b_{res}^r) from 5.8 to 7.5×10^{-4}

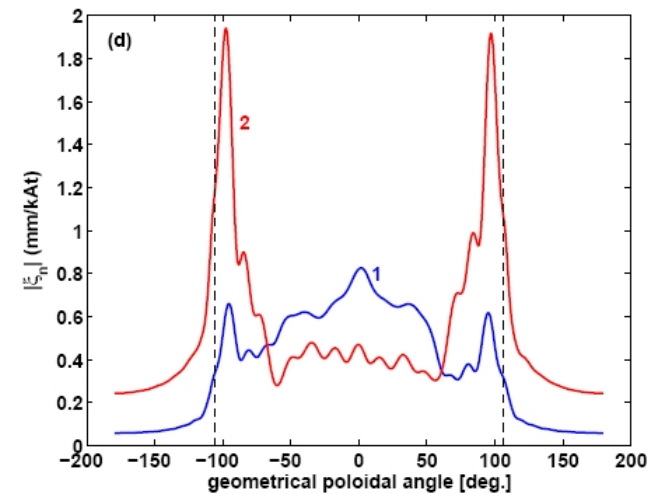
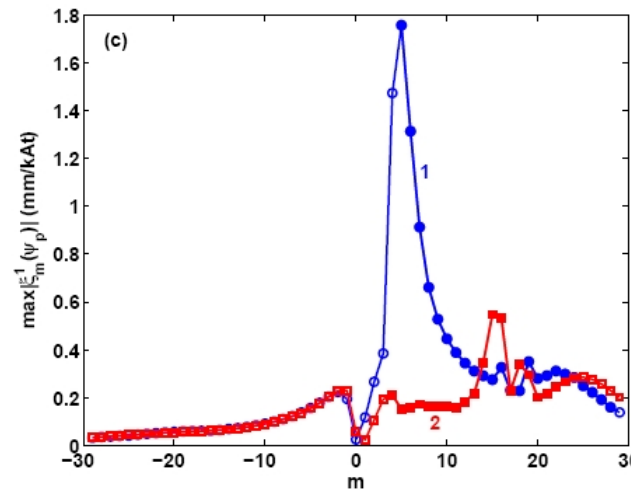
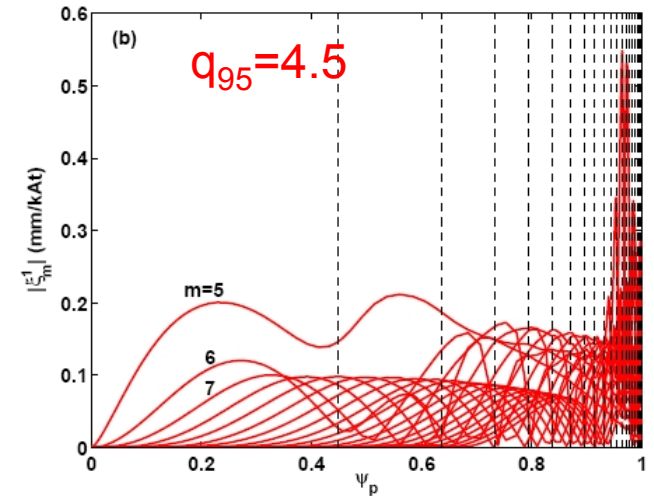
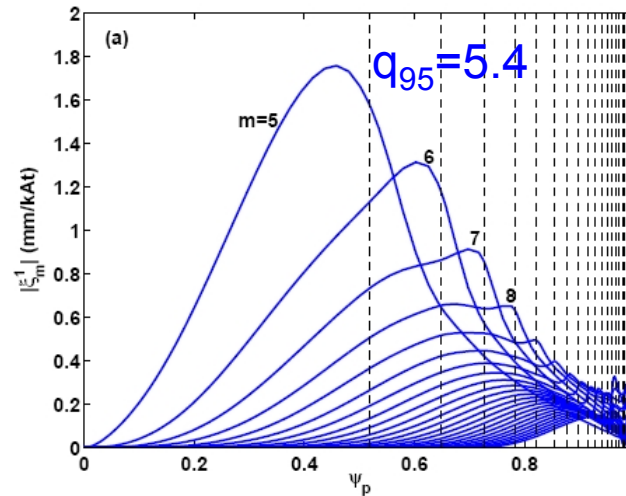
Plasma displacement at the X-point also becomes more dominant (MARS-F)

X-point peaking and peeling mode

$q_{95} = 5.4$ compared to 4.5 for odd parity

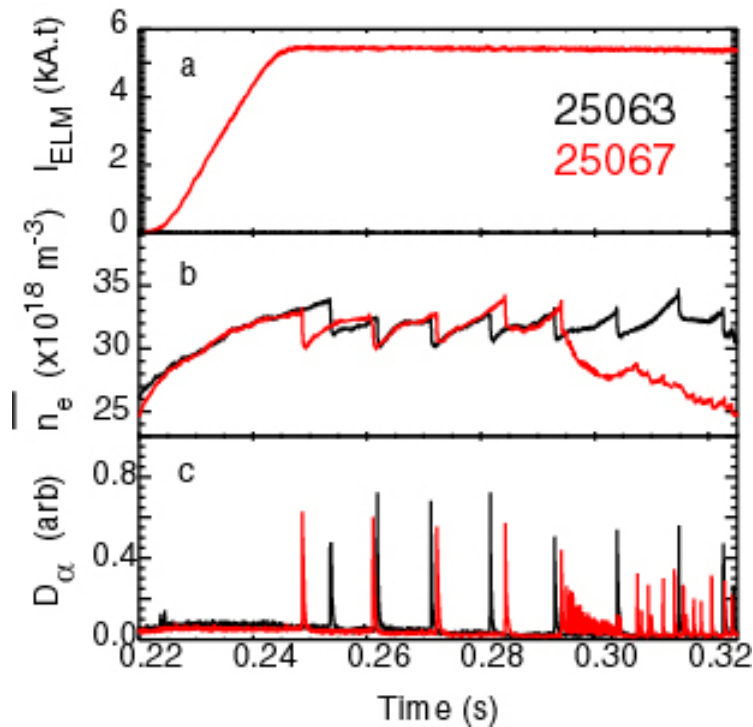


Plasma displacement
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X-point peaking again associated with peeling mode becoming dominant

Effect of edge ν_e^*

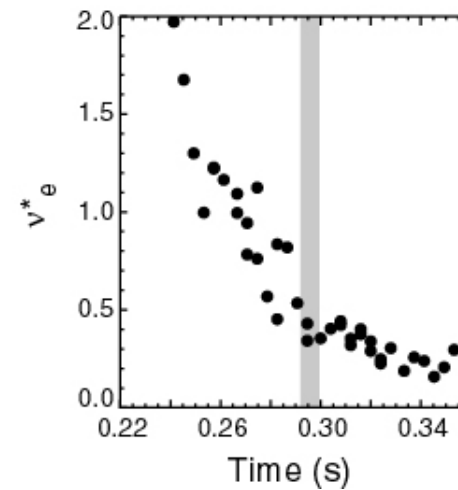


Get clear pump out and effect on ELMs

f_{ELM} increases by 5

ΔW_{ELM} reduces from 5 kJ to ~ 1kJ

W_{MHD} reduces by ~ 8%

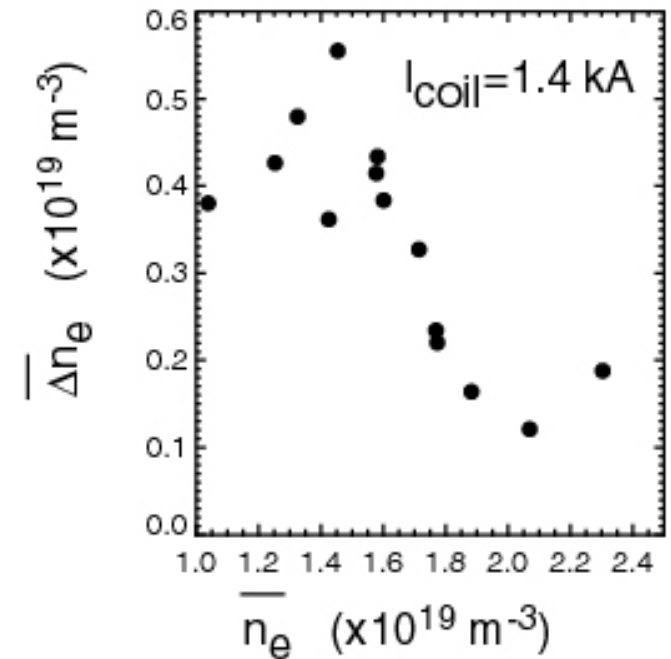


Pump out occurs when $\nu_e^* < 0.5$

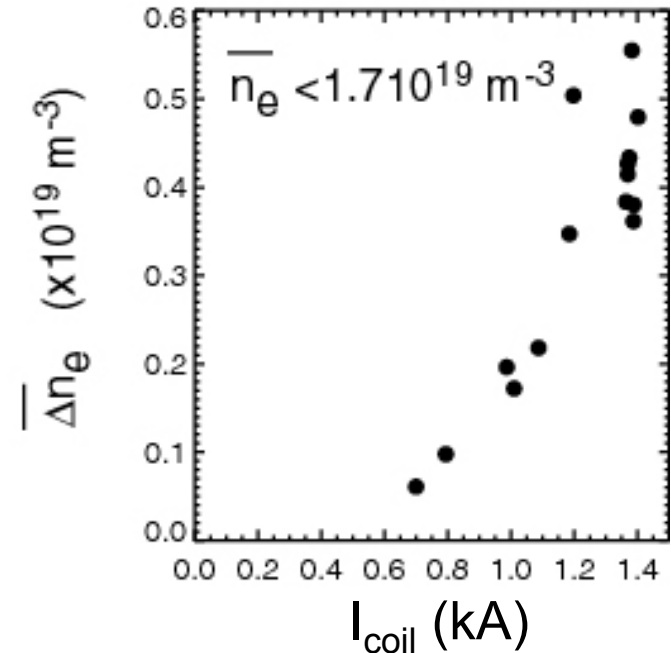
L-mode results

➤ Pump-out trend with n_e and I_{coil}

- At full coils current (1.4kA), the pump-out decreases with density



- At moderate density ($< 1.7 \times 10^{19} \text{ m}^{-3}$), the pump-out increases linearly with I_{coil}
 - Offset suggests threshold



At least three different flavors of ELM control exist

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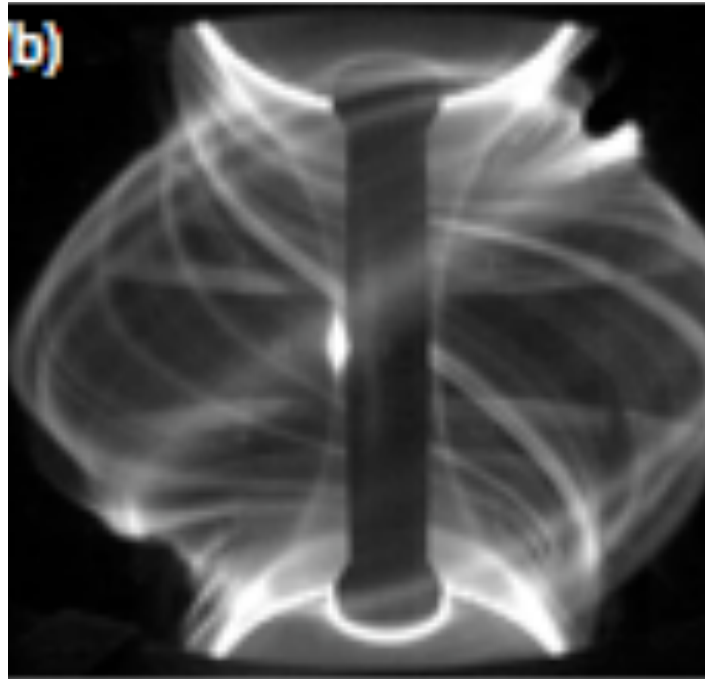
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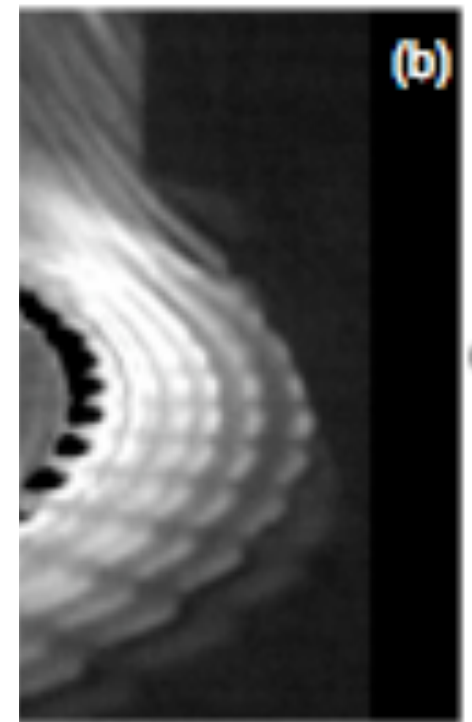
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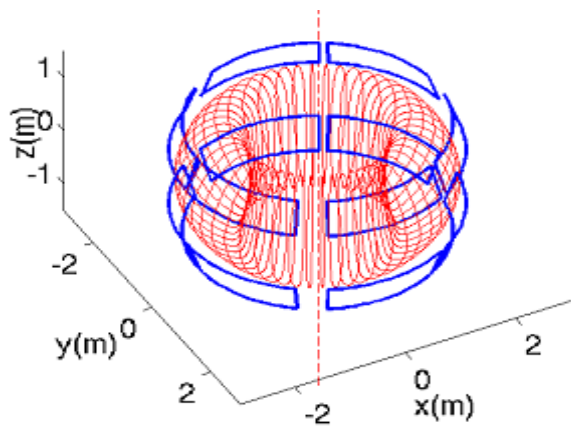
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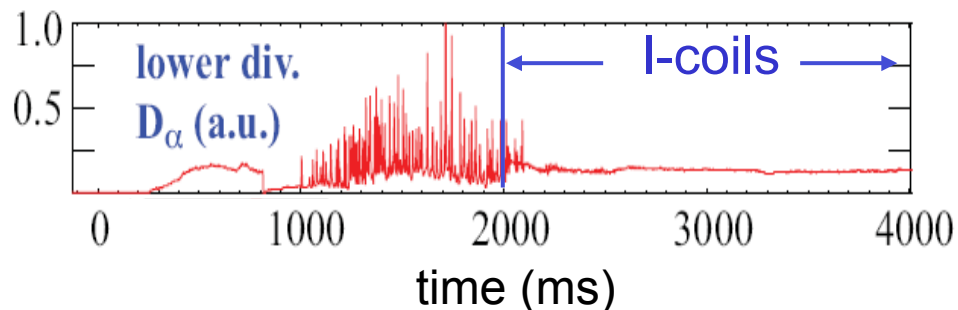
Background and motivation

- Type I ELMs are expected to be a factor 20 too large in ITER
- A candidate solution: Resonant Magnetic Perturbations (RMPs)
- Promising results on different machines:

ELM suppression on DIII-D

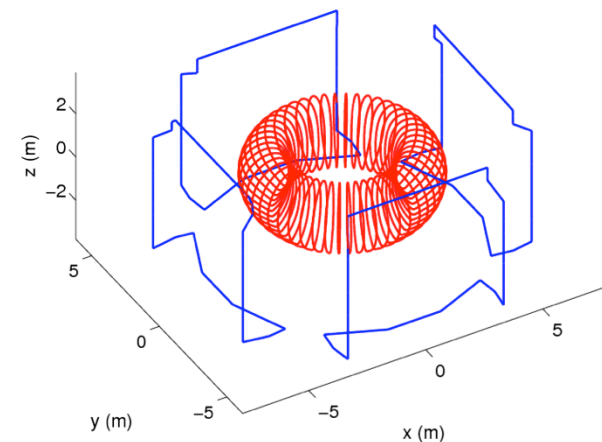


I-coils: 2 rows of 6 in-vessel coils, $n=3$

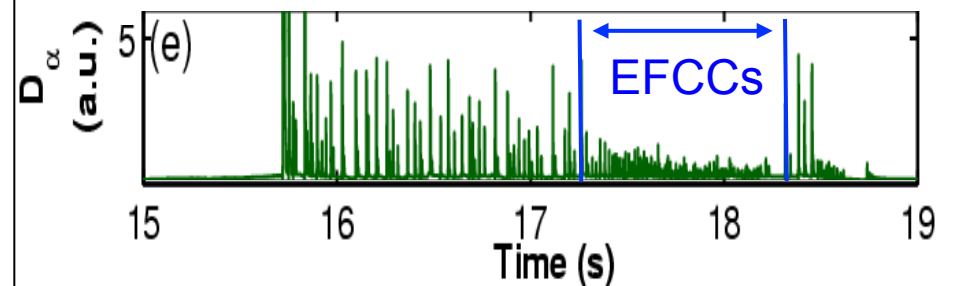


ELM mitigation on JET

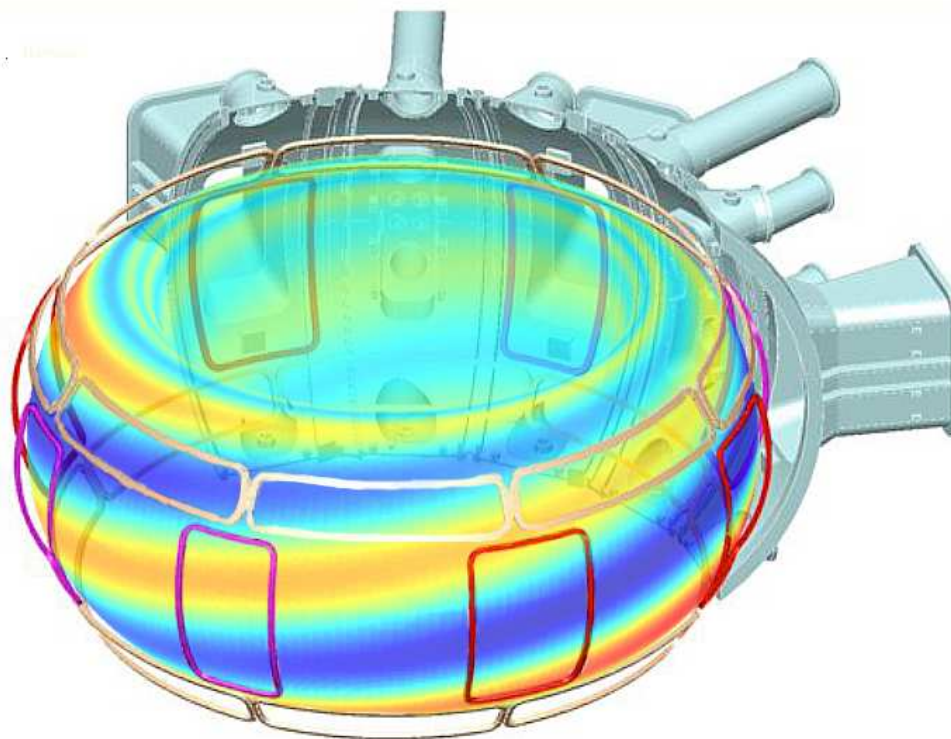
Y. Liang, O5.062 on Friday



EFCCs: 4 coils, ex-vessel, $n=1$ or 2



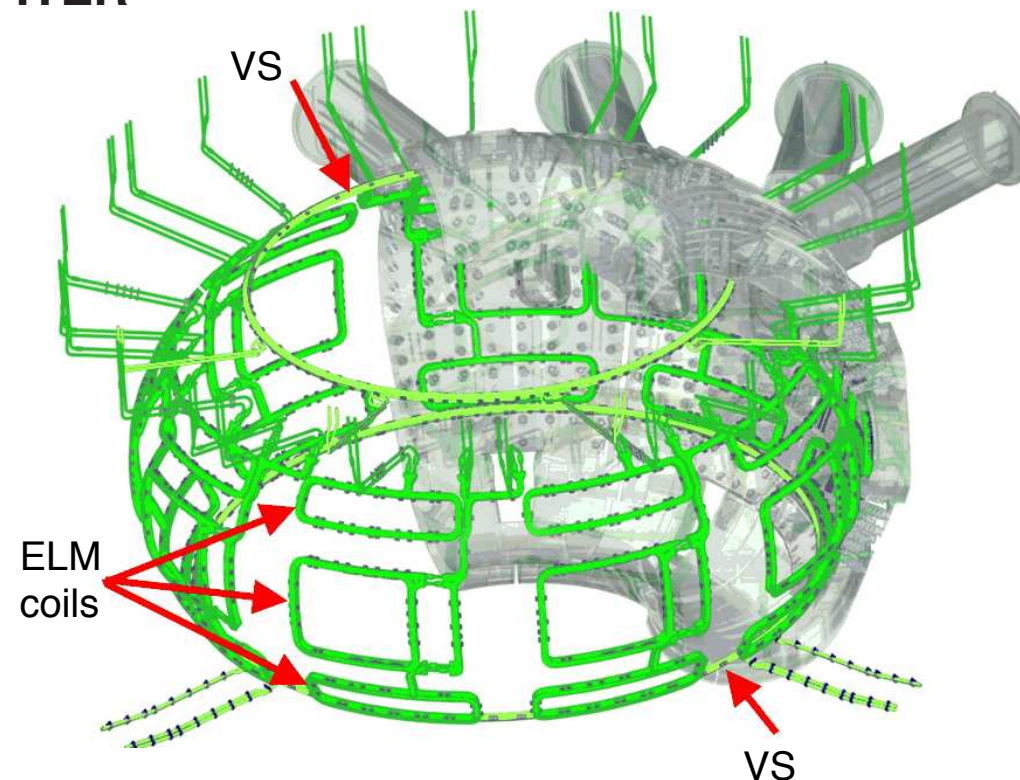
ASDEX Upgrade



3 rows à 8 saddle coils
in-vessel, low field side
individual current feeds, $n \leq 4$

W Suttrop *et al*, Fus. Eng. Des. 84 (2009) 209

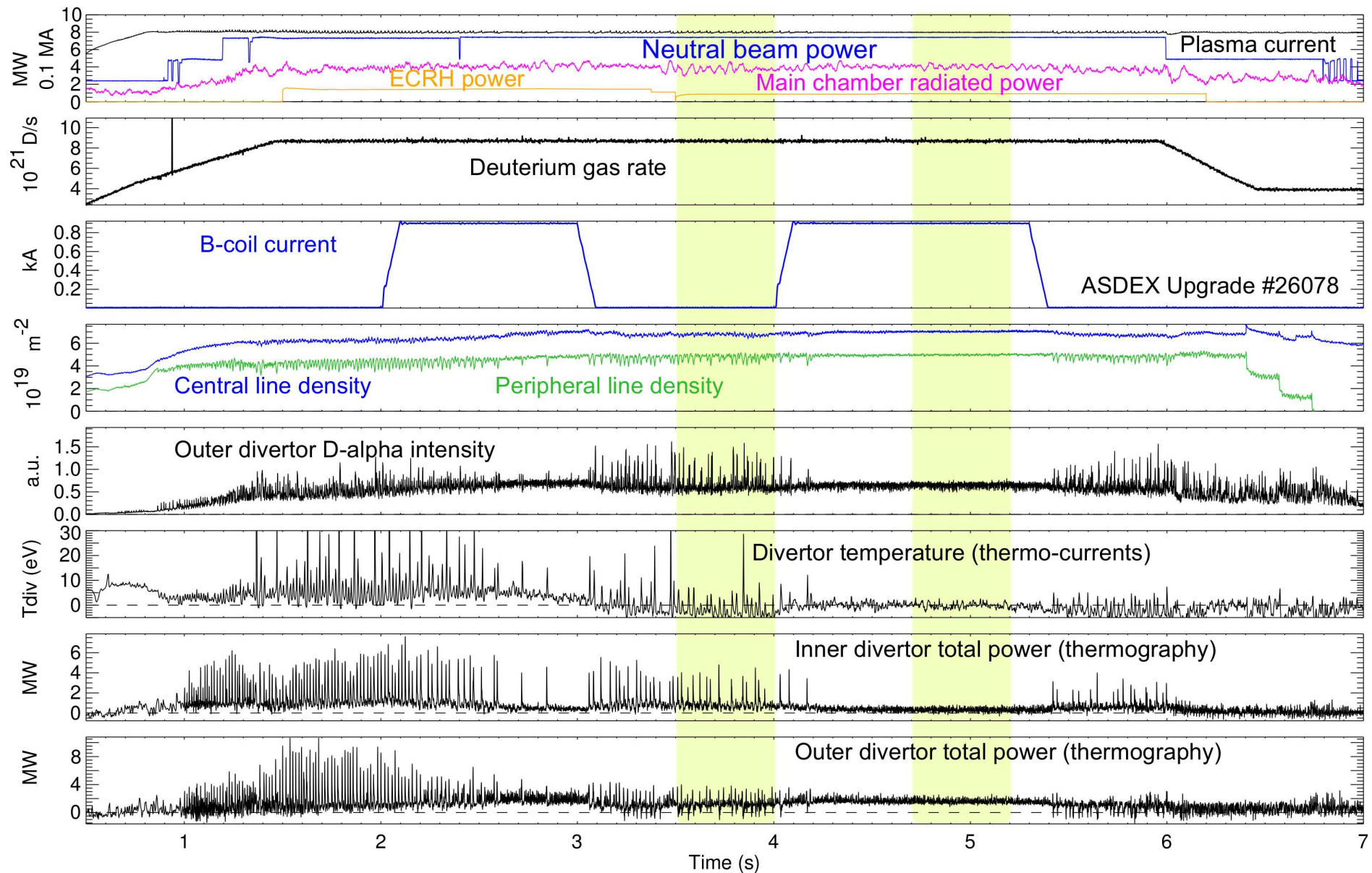
ITER



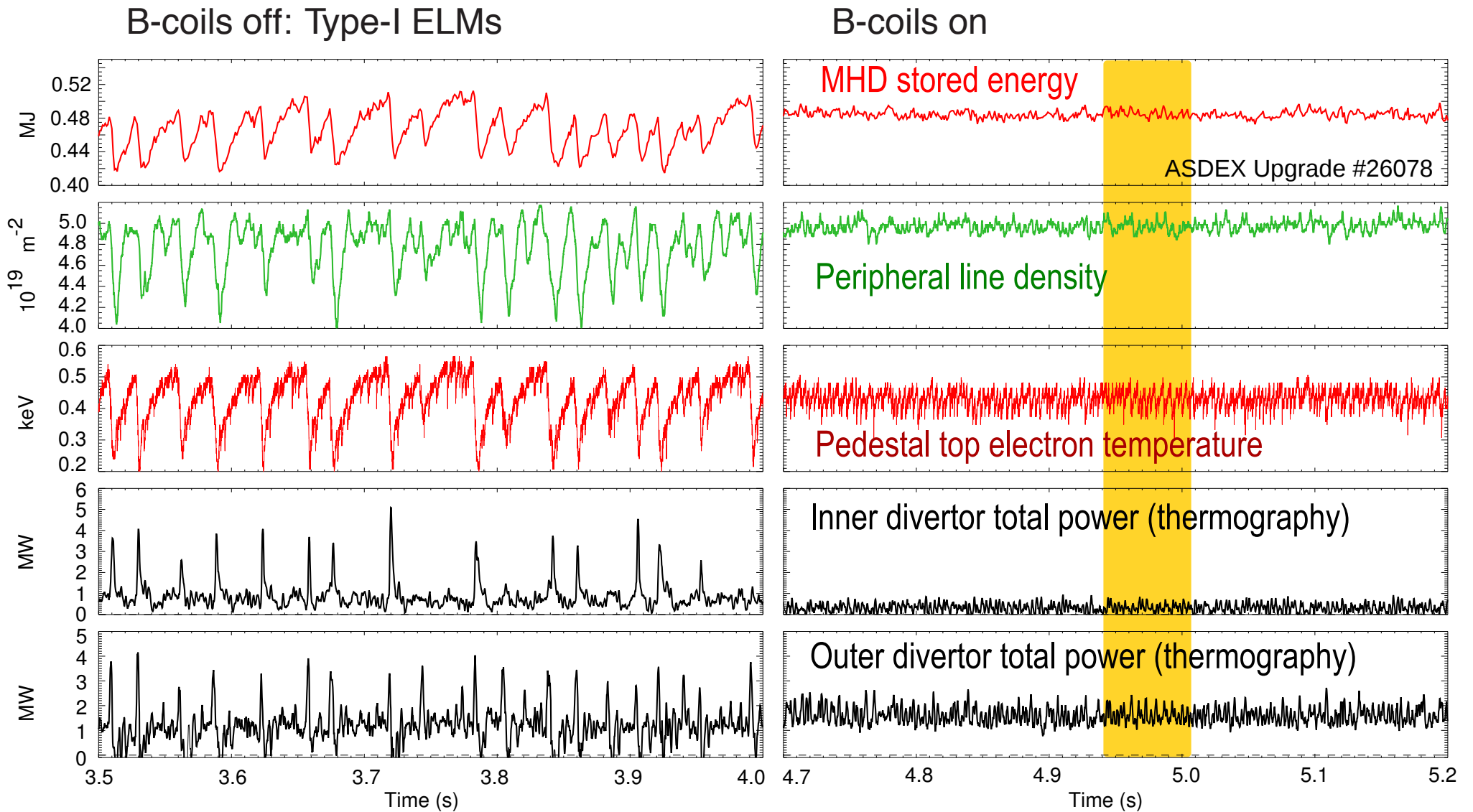
3 rows à 9 saddle coils
in-vessel, low field side
individual current feeds, $n \leq 4$

A Loarte, ITPA PEP March 2011

ELM-mitigation observed as B-coil currents applied



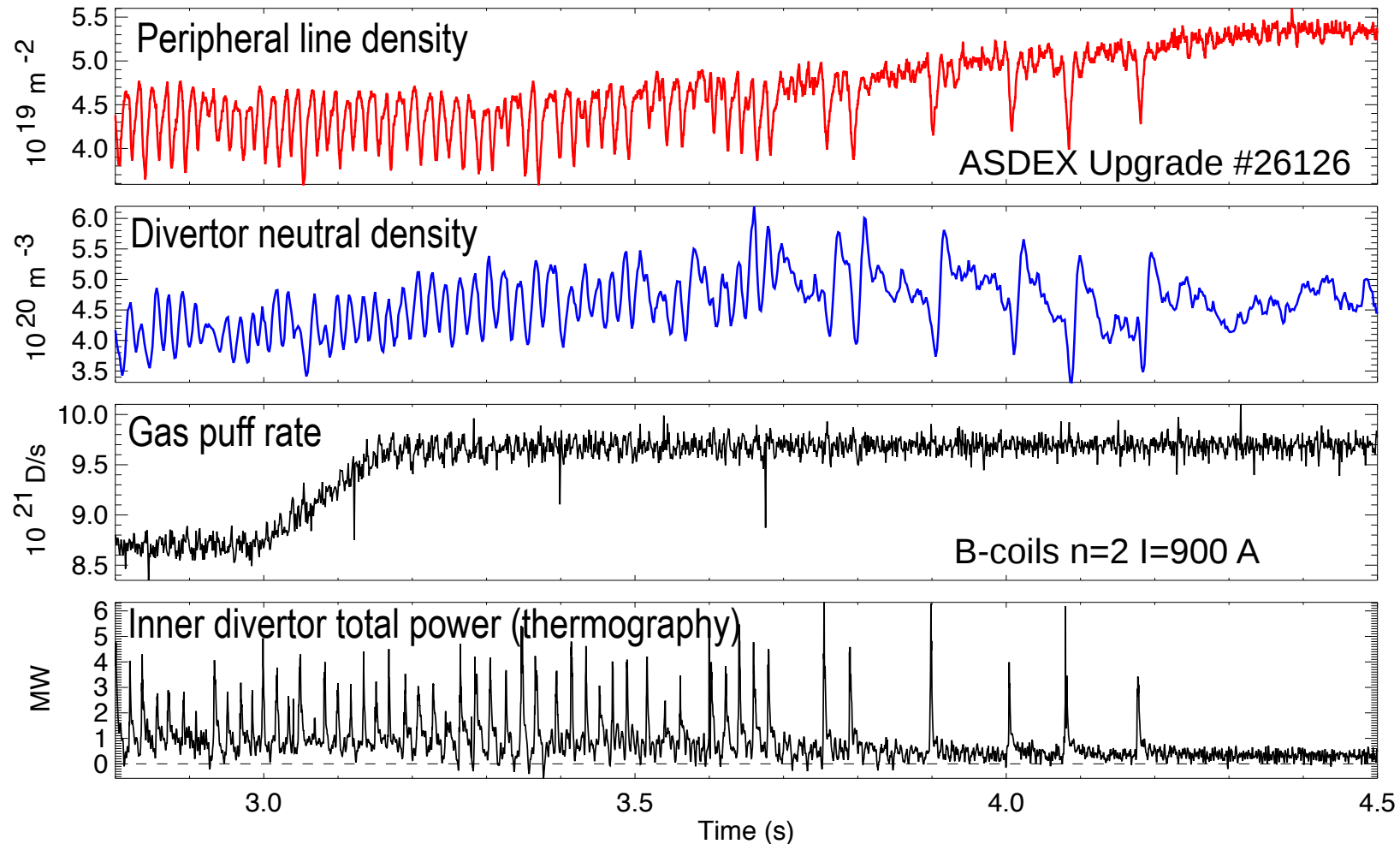
Comparison of Type-I ELMy and ELM-mitigated phases



Much reduced excursions of W_{MHD} , \bar{n}_e , $T_{e,\text{ped}}$, P_{div} !

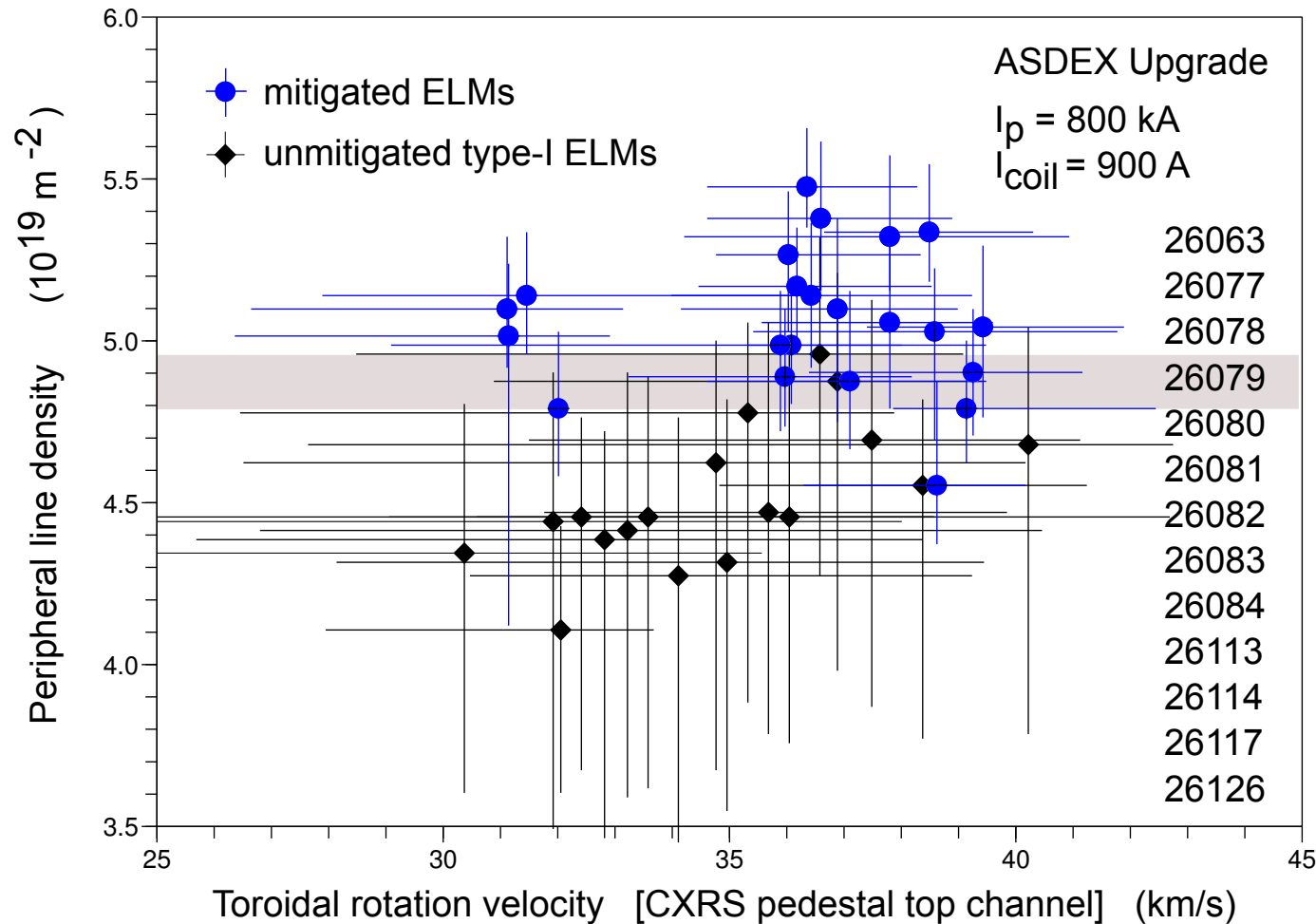
Particle confinement increases as type-I ELMs are suppressed

Slow transition induced by increasing plasma density:



- Plasma density rises while neutral density drops
- Minimum plasma density required for type-I ELM suppression

Plasma rotation varied by NBI momentum input + NBI / ECRH mix



ELM mitigation with all probed heating scenarios: $P_{NBI} = 2.5 - 12.5$ MW, dominant RF.

Large type-I ELMs replaced by small, irregular ELMs; reduced heat and particle losses.

- ELM-type transition, not a gradual evolution of ELM losses
- Power load reduction factor mainly determined by type-I ELM losses to compare with
- No price in confinement, density, impurity content
- Resembles DIII-D high collisionality regime [Evans et al. 2005]

Access conditions:

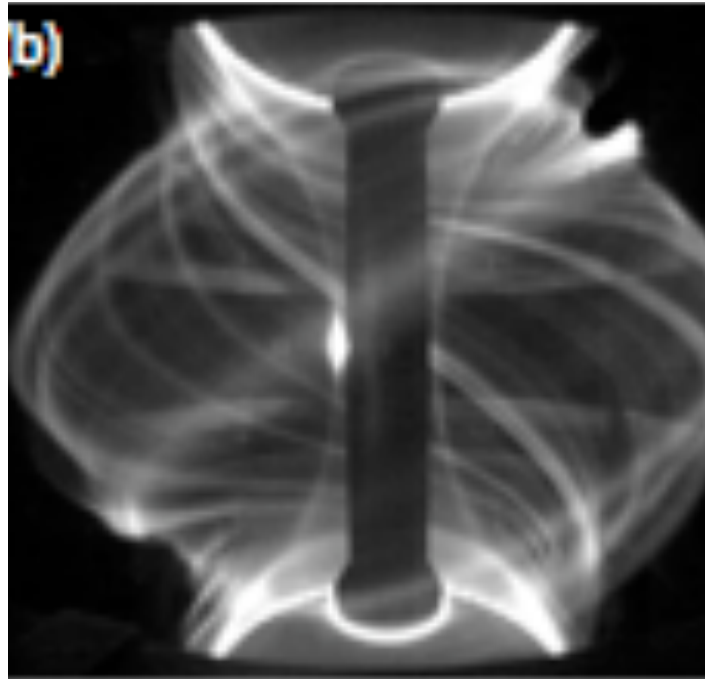
- **Insensitivity to magnetic resonance**
- **Density threshold**, depends on I_p ; ($n_{GW} = 0.65$, $\nu^* \approx 1.4$)
still to disentangle which parameter critical

Outlook:

- Requirements for low collisionality ELM suppression/mitigation
- Broaden experimental data base, continue comparison with JET, DIII-D, MAST, NSTX
- Installation of additional eight coils 3+4.Q 2011, experiments 2012–

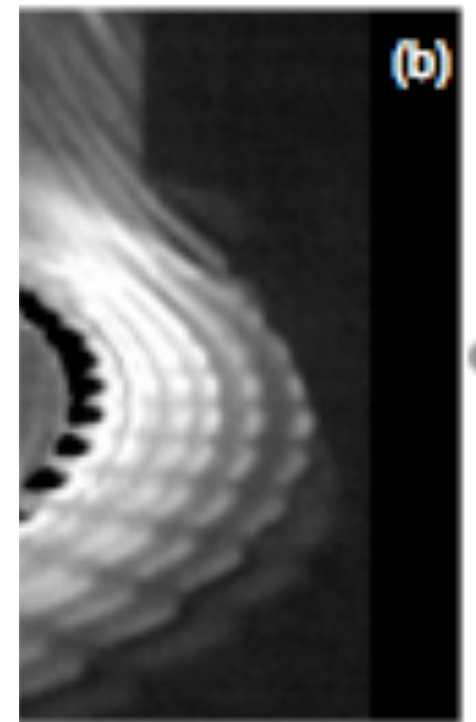
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 - Used to control impurity transport in ELM-free regimes

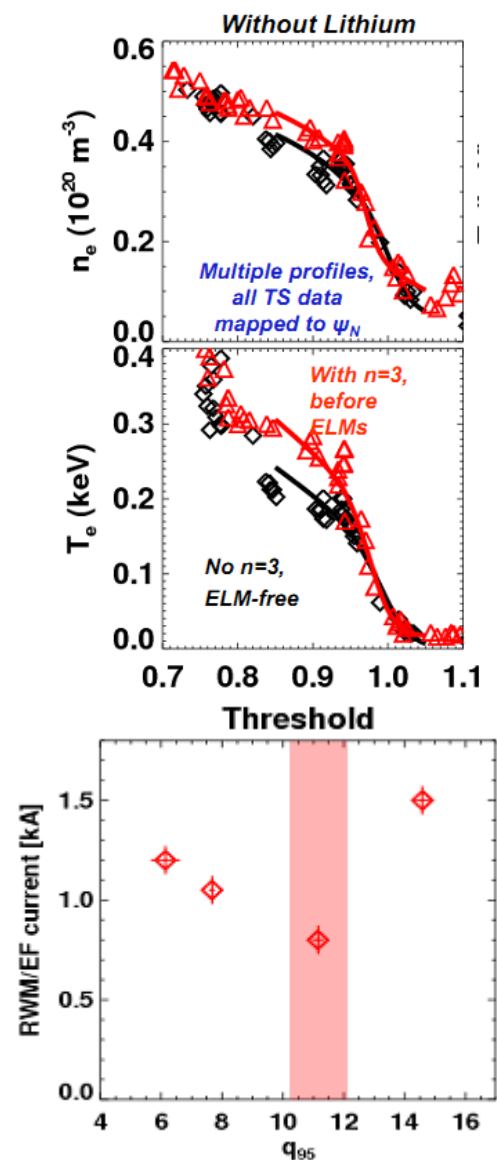


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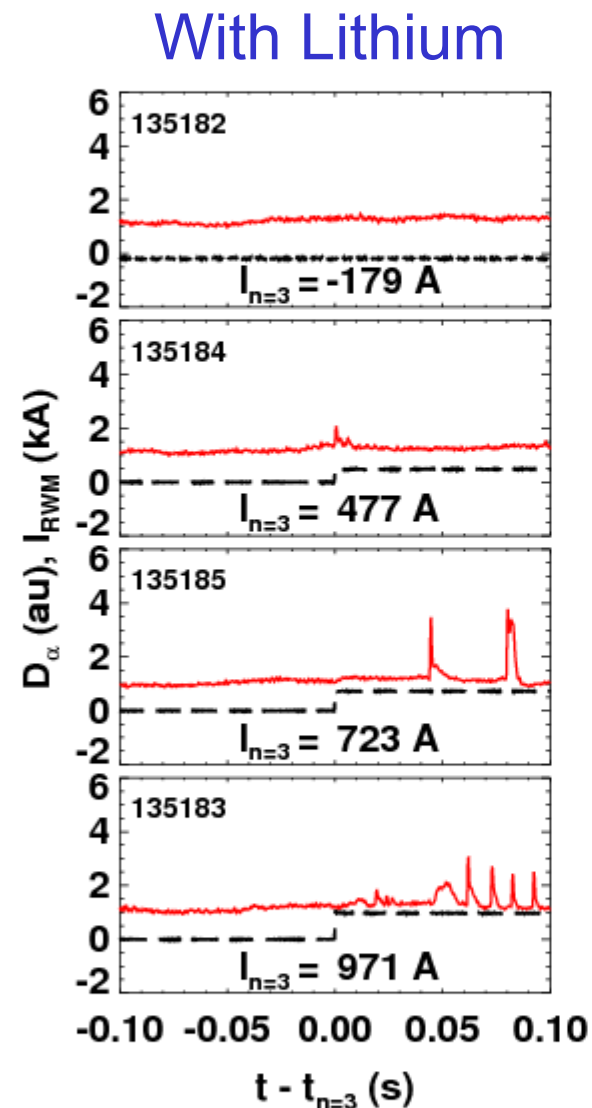
*Recycling profile on
target plates*



In NSTX, ELMs are destabilized above a threshold perturbation when DC fields are applied



- Above a threshold, $n=3$ field destabilizes ELMs without and with lithium-coated PFCs
 - ELM frequency increases with $n=3$ field magnitude
- $n=3$ field has little effect on n_e , sometimes (not always) increases T_e
- Threshold 3D field for destabilization shows q_{95} dependence
 - Optimum window for ELM triggering at $q_{95} \sim 11$



Magnetic ELM triggering has been applied to lithiumized ELM-free H-modes to control impurity accumulation

Typical behavior with Li wall conditioning

ELMs suppressed

P_{rad} ramps to >2 MW; $P_{\text{NBI}} = 3$ MW

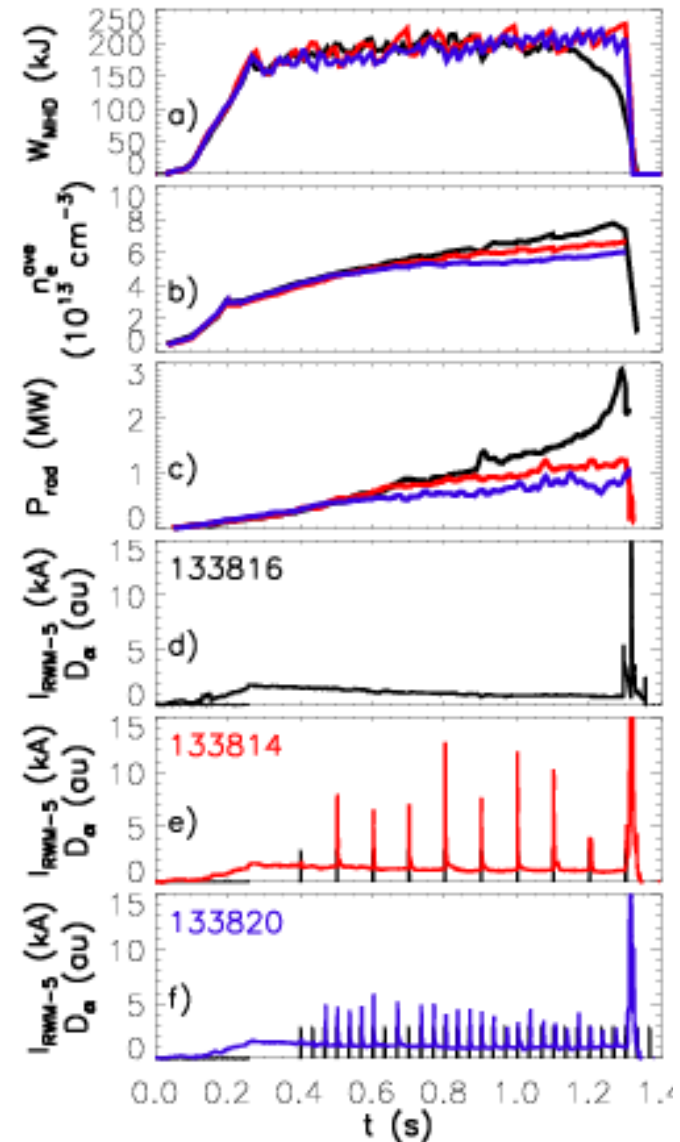
Square wave of $n=3$ fields applied

Fast pulses used rather than DC fields to reduce rotation braking

4 ms pulses, $f=10/30$ Hz, amp. 2.2 kA

ELMs can be triggered with full control over timing and frequency

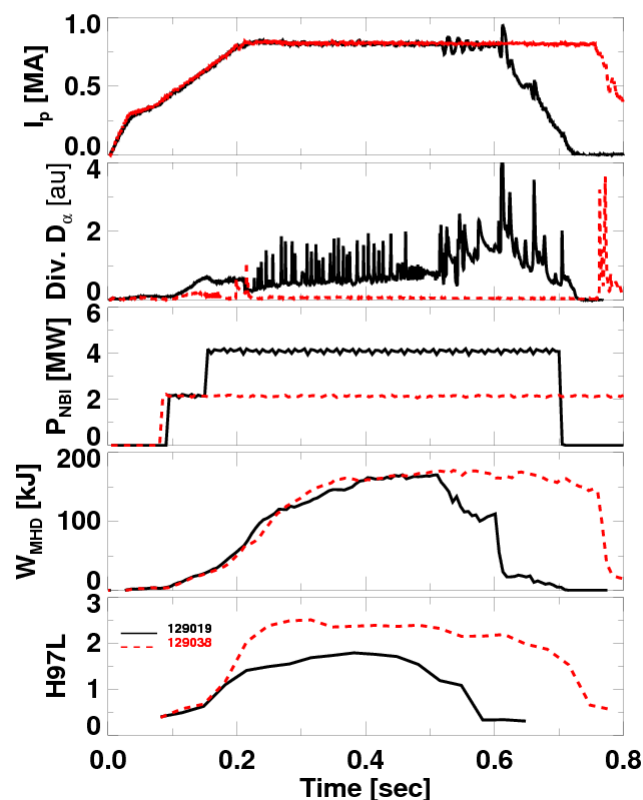
Used here for discharge control, reducing n_e and P_{rad} ramp rate



J.M. Canik et al, PRL 104, 045001 (2010)

NSTX lithium coating experiments highlight the potential for controlling ELMs through the pedestal density profile

- With lithium, edge n_e gradient is reduced
 - Reduces pressure gradient/bootstrap current, eliminates ELMs*
- Indicates that controlling pedestal density profile can be used to control ELM behavior
 - Similar to DIII-D RMP experiments)



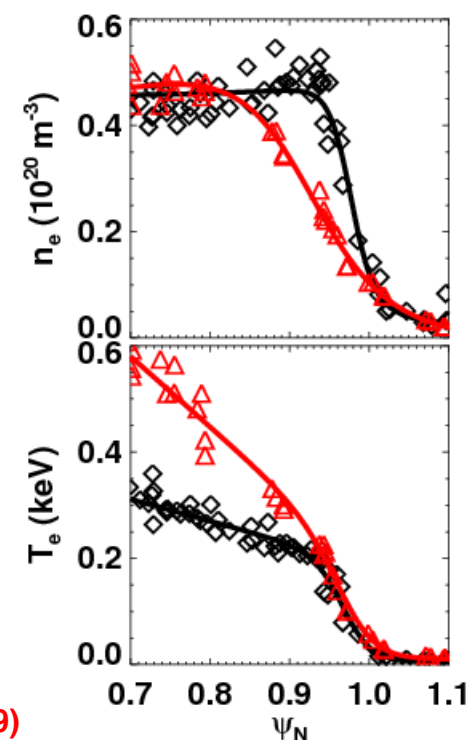
Without Li, **With Li**

**ELM-free, reduced
divertor recycling**

**Lower NBI to avoid β
limit**

Similar stored energy

H-factor 40% \uparrow



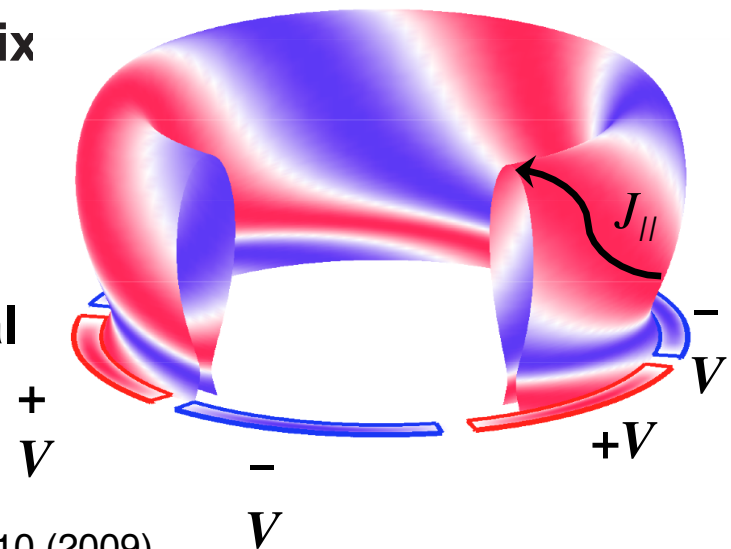
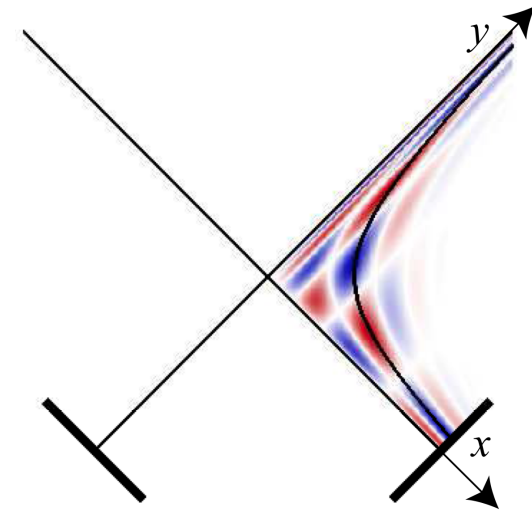
*R. Maingi et al, PRL 103, 075001 (2009)

Selected ELM control references

- **ASDEX-Upgrade**
 - W. Suttrop, et al., Phys. Rev. Lett. **106** (2011) 225004
- **DIII-D**
 - T.E. Evans, et al., Phys. Rev. Lett. **92** (2004) 235002
 - K.H. Burrell, et al., Plasma Phys. Control. Fusion **47** B37 (2005)
 - T.E. Evans, et al., Nature Phys. **2** 419 (2006)
- **JET**
 - Y. Liang, et al., Phys. Rev. Lett. **98** (2007) 265004
 - Y. Liang, et al., Nucl. Fusion **50** (2010) 025013
- **MAST**
 - E. Nardon, et al., Plasma Phys. Control. Fusion **51** (2009) 124010
 - A. Kirk, et al., Nucl. Fusion **50** (2010) 034008
 - A. Kirk, et al., Plasma Phys. Control. Fusion **53** (2011) 065011
- **NSTX**
 - J.M. Canik, et al., Phys. Rev. Lett. **104** (2010) 045001
 - J.M. Canik, et al., Nucl. Fusion **50** (2010) 034012
- **Recent Review (does not cover new AUG results)**
 - Y. Liang, Fusion Sci.Tech. **59** (2011) 586

For a reactor, **challenging in-vessel coil designs** can be replaced by **non-axisymmetric scrape-off layer current**

- Non-axisymmetric variations in the electrostatic potential drive both $\mathbf{E} \times \mathbf{B}$ convection and parallel current J_{\parallel}
- **SOL convection**¹ can be used to spread particle and heat fluxes in the divertor
- **SOL current**² can be used to generate magnetic perturbations inside the separatrix that controls pedestal transport & stability
- Can be driven either by direct electrical biasing or by passive generation of toroidal divertor asymmetries



¹R. H. Cohen and D. D. Ryutov, Nucl. Fusion **37** 621 (1997)

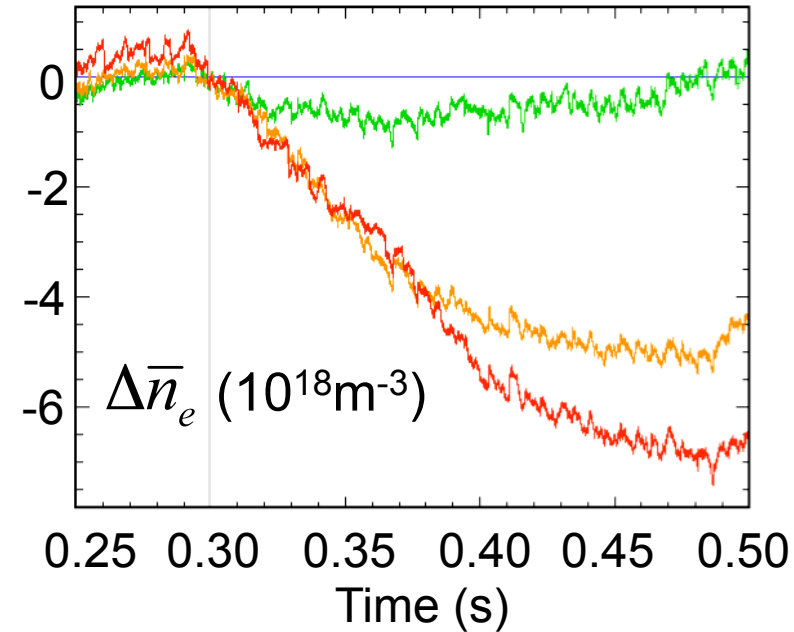
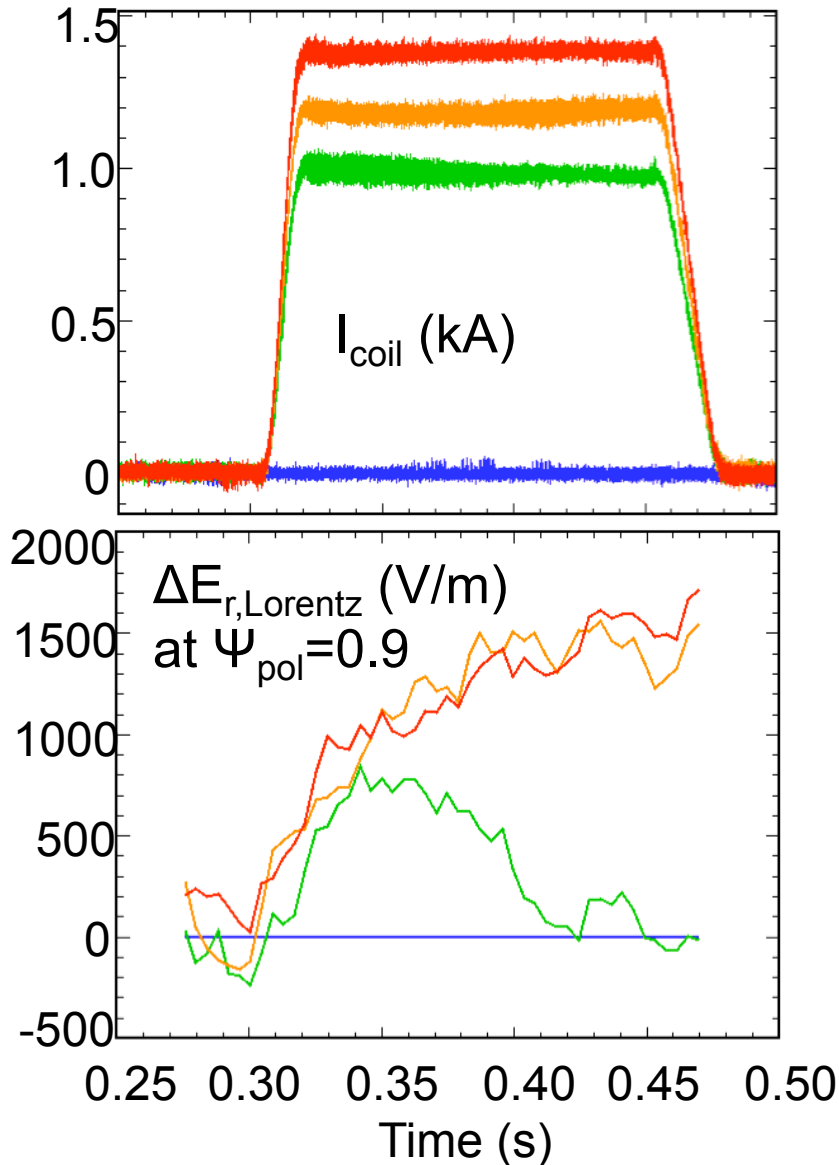
²I. Joseph, R. H. Cohen and D. D. Ryutov, Phys. Plasmas **16** 052510 (2009)

Additional Material ...

L-mode results

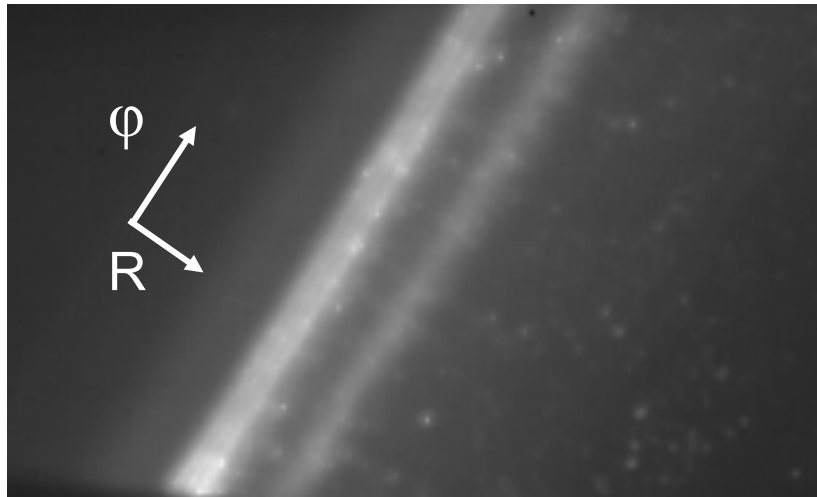
➤ Visible increase in E_r due to its Lorentz part measured by Doppler spectroscopy on He: $E_{r,Lorentz} = v_{\theta,He} B_{\phi} - v_{\phi,He} B_{\theta}$

D. Temple, P5.191 on Friday

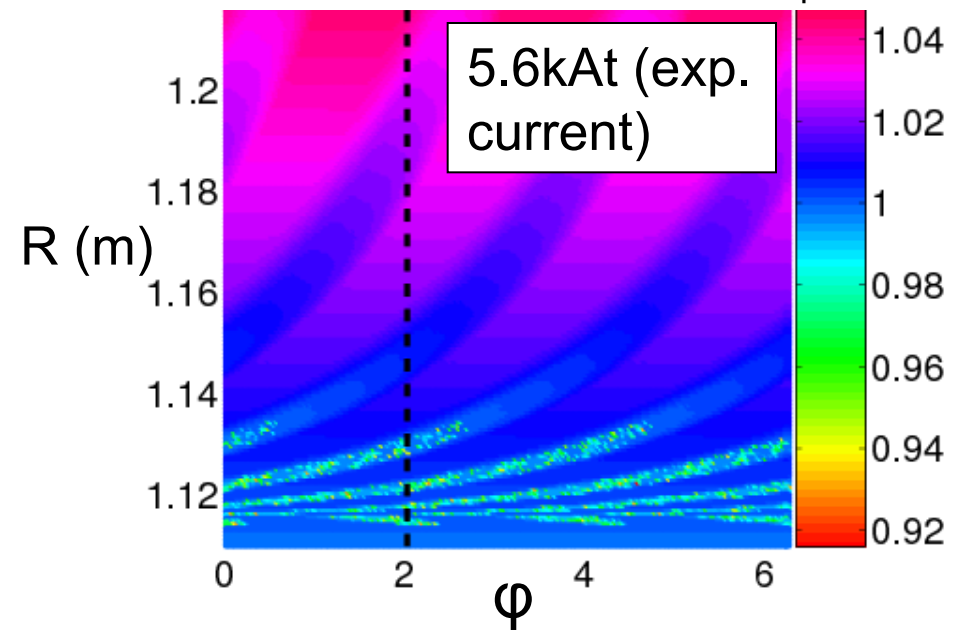


- $\Delta E_r \sim 1.5 kV/m$
- $\Delta E_r > 0$ is expected in a stochastic field to preserve ambipolarity
- DIII-D with the I-coils (Burrell '05) and TEXTOR with the DED (Unterberg '07) have $\Delta E_r \sim 10 kV/m$

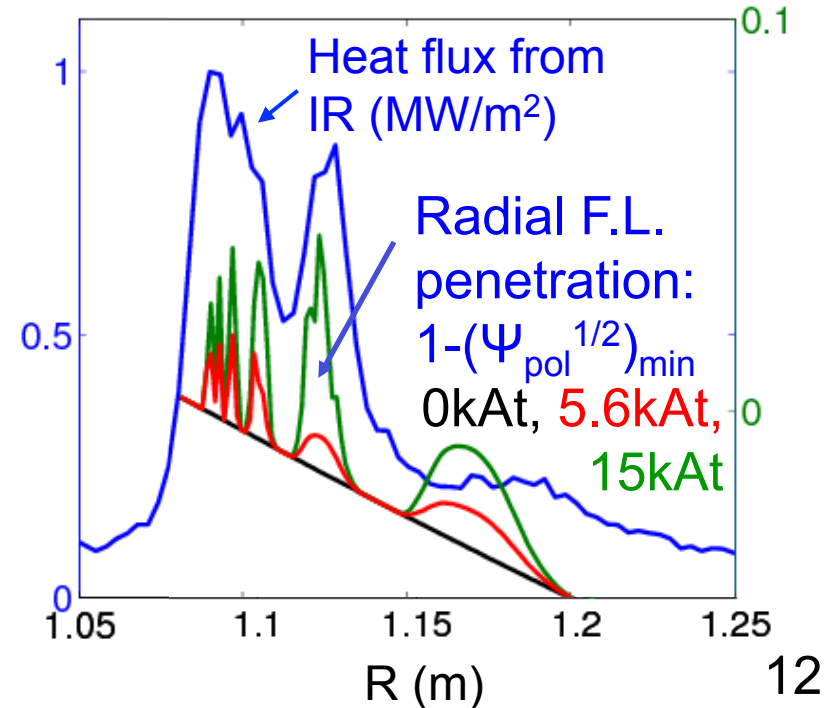
L-mode results



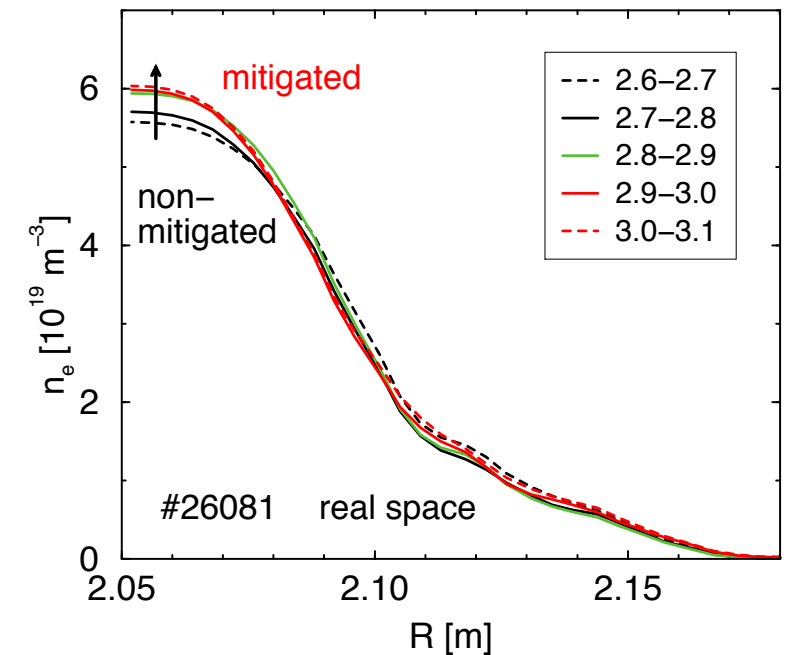
Deepest radius reached by FL: $(\Psi_{\text{pol}}^{1/2})_{\text{min}}$



- Strike point splitting observed on infrared images of the outer target plate
- Vacuum modelling predicts a splitting
 - In spite of stochasticity, a coherent spiraling magnetic footprint exists
- Good match between measured heat flux profile and calculated field lines radial penetration



- Type-I ELMs replaced by frequent small ELMs (not a gradual evolution of ELM losses)
- Divertor peak power reduction —
Inner: Continuous detachment
Outer: up to factor 4 (steady + ELM pulse)
- Particle confinement, pedestal density increase
- Pedestal temperature reduced by $\approx 10\%$
- Confinement / stored energy essentially unchanged
- Heavy impurity concentration (W!) not increased
- Z_{eff} not increased
- Minimum density requirement for ELM mitigation
- D pellet fueling: $n \approx 1.5 \times n_{\text{Greenwald}}$,
no ELMs triggered



Pedestal profiles, Z_{eff} :

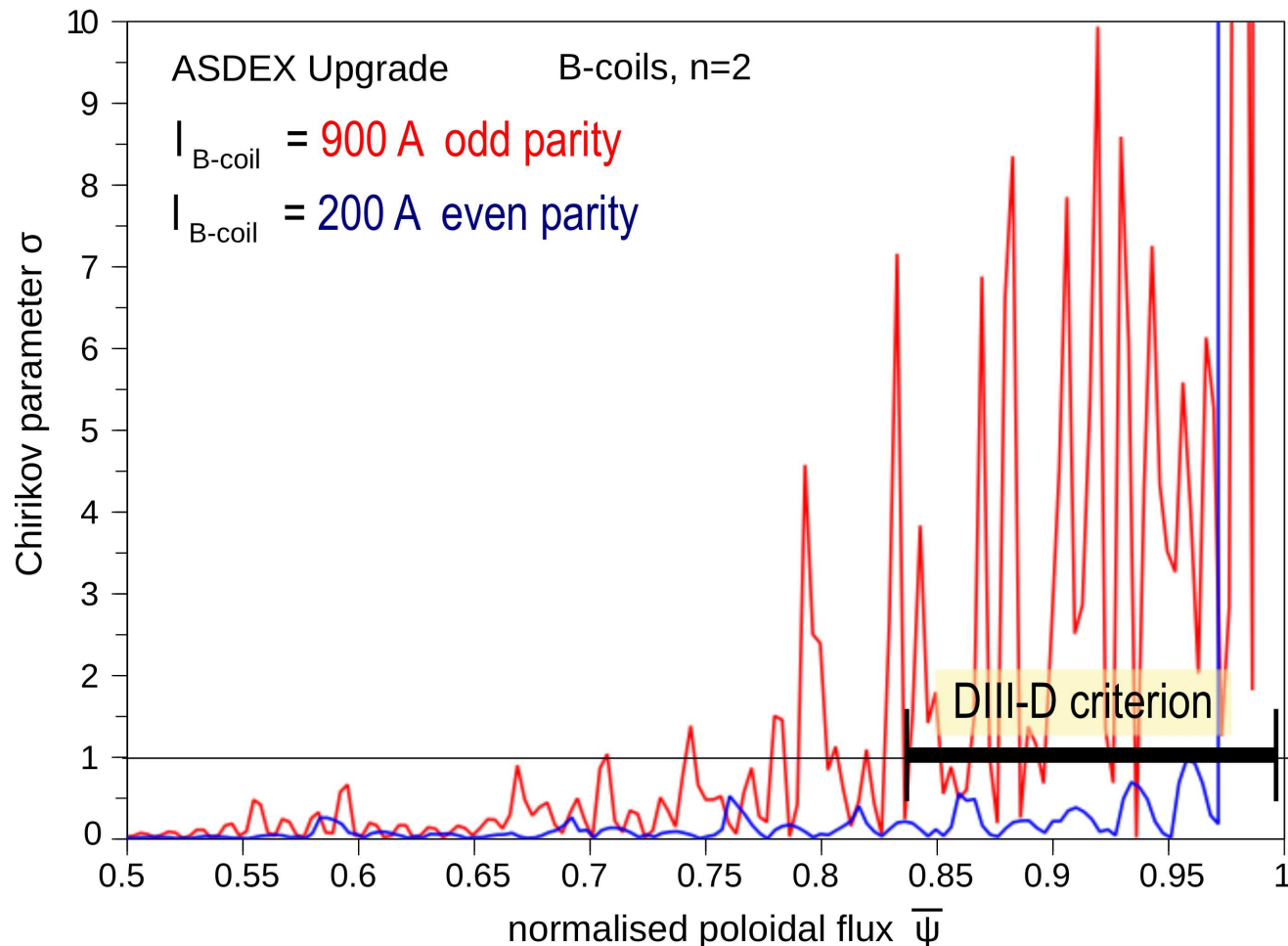
R Fischer, P1.072

B Kurzan, P4.048

Pellet fueling:

P T Lang, O.3.112

GOURDON field line tracing



$$\sigma = \frac{(\psi - \psi_0)_{\max}}{\psi_{m+} - \psi_{m-}}$$

Ergodic field where $\sigma > 1$

DIII-D criterion:

$\sigma > 1$ for $\bar{\psi} > 0.83$

[Fenstermacher et al. 2008]

ELM mitigation also with:

$\sigma > 1$ *only* for $\bar{\psi} > 0.97$

(even parity)

N.B.

Close to separatrix:

— $(\psi_{m+} - \psi_{m-}) \rightarrow 0$

— open field lines
always resonant

Chirikov parameter $\sigma > 1$ not a sufficient condition

Low q_{95} , low density attempt; Type-I ELMy H-mode

